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VIRGINIA ELECTRIC AND POWER COMPANY, RICHMOND, VIRGINIA 23261

June 20, 1979

Mr. James P. O'Reilly, Director
Office of Inspection & Enforcement
U. S. Nuclear Regulatory Commission
Region II
101 Marietta Street, Suite 3100
Atlanta, Georgia 30303

Serial No. 409A
PSE&C/GLS:mac:wang

Docket No. 50-339

Dear Mr. O'Reilly:

On May 21, 1979, a report was made under the provisions of 10CFR50.55(e) concerning overweight motor operated valves on the accumulator discharge lines. This was followed by a five day report on May 24, 1979, under the provisions of 10CFR21 stating that the actual weight of the three 12" motor operated valves (one on the discharge of each of the three accumulators in the safety injection system) is greater than that used by Stone & Webster in their original pipe stress analysis. It was postulated that this error could result in overstressed piping under certain seismic loading conditions.

Stone & Webster has completed the review of the effect of changing the weight of the Velan/Westinghouse supplied valves (identified as 12GM58FJH) from 3050 pounds to 3950 pounds. The task required a reanalysis of three Class 1 pipe stress problems and a review of the revised loadings on the pipe supports, reactor coolant loop nozzles, accumulator tank nozzles, and the accumulator tank supporting structure.

All pipe stresses are within the applicable ASME allowables.

The revised reactor coolant loop nozzle loads and accumulator tank nozzle loads were transmitted to Westinghouse and determined to be acceptable. The existing pipe support designs were reviewed using the new loads. As a result, it was necessary to reset three of the spring hangers. This field work has been completed, and no other modifications to existing supports were required. The accumulator tank supports, an S&W design, were also reviewed and determined to be adequate.

The lines involved were originally subjected to a pipe break analysis. Each line was rechecked based upon our FSAR docketed approach which determined break locations by stress level. This rupture review indicated that no additional break points were created and no modification of existing restraints was necessary.

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This report shall be considered our final report under the provisions of 10CFR50.55(e). Should you require further information, please contact this office.

Very truly yours,

Sam C. Brown, Jr.
Senior Vice President - Power Station
Engineering and Construction

cc: Mr. John G. Davis, Acting Director
Office of Inspection & Enforcement

Mr. Harold R. Denton, Director
Office of Nuclear Reactor Regulation

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