



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

MAY 15 1979

Docket No. 50-348

LICENSEE: Alabama Power Company (APCO)  
FACILITY: Farley Nuclear Plant, Unit No. 1  
SUBJECT: SUMMARY OF MEETING HELD ON MAY 8, 1979 TO DISCUSS  
THE AUXILIARY FEEDWATER SYSTEMS AND RELATED AREAS

On May 8, 1979 representatives of APCO met with the Regulatory Staff in Bethesda, Maryland to discuss the auxiliary feedwater system and related station information. The purpose of the meeting was to assemble information for an NRC staff meeting report. The areas discussed are listed in enclosures 1 and 2. Meeting attendees are listed in enclosure 3.

APCO personnel presented information to the NRC staff for discussion. Certain plant drawings and existing Final Safety Analysis Report sections were reviewed. Operating experience and the most current operational, test and maintenance procedures were presented and discussed. APCO noted that Farley Unit No. 1 was shutdown on March 8, 1979 for start of Cycle 2 fueling. Scheduled startup is late May or early June, 1979.

APCO (Epps) noted that their company policy was to maintain all current operational procedures at the plant site. However, APCO provided copies of the latest versions (some still under review by APCO staff) of procedures identified in enclosure 1 for review during the meeting. Other material presented to the NRC staff by APCO will be held by the NRC Project Manager pending completion of the NRC staff report.

By enclosure 2 we requested APCO to provide written responses at the meetings. Due to the short notice prior to the meeting, APCO presented the information in preliminary form for information only. We accepted the data presented for use as background information and agreed to contact APCO (T. N. Epps) if further data was needed.

A handwritten signature in cursive script that reads "Edward A. Reeves".

Edward A. Reeves, Project Manager  
Operating Reactors Branch #1  
Division of Operating Reactors

Enclosures:  
As stated

cc: w/enclosuressee attached list

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Docket Files

NRC PDR

Local PDR

ORBI Reading

NRR Reading

H. Denton

E. Case

V. Stello

D. Eisenhut

B. Grimes

R. Vollmer

A. Schwencer

D. Ziemann

P. Check

G. Lainas

D. Davis

B. Grimes

T. Ippolito

R. Reid

V. Noonan

G. Knighton

D. Brinkman

Project Manager

OELD

OI&E (3)

C. Parrish

ACRS (16)

NRC Participants

J. Buchanan

TERA

Licensee

Short Service List

P. Mathews

L. Kintner

T. N. Epps, APC

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Southern Services, Inc.  
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John Bingham, Esquire  
Balch, Bingham, Baker, Hawthorne,  
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Edward H. Keiler, Esquire  
Keiler and Buckley  
9047 Jefferson Highway  
River Ridge, Louisiana 70123

George S. Houston Memorial Library  
212 W. Berdeshaw Street  
Dothan, Alabama 36303

As part of its on-going review of the Three Mile Island Unit 2 accident, the staff finds that it needs additional information regarding the auxiliary feedwater system (AFWS). This information as outlined below, is required to evaluate AFWS reliability for Combustion Engineering (CE) and Westinghouse (W) designed pressurized water reactors. The requested information is in addition to that requested in the IE Bulletins, and should be brought to the meeting scheduled with the staff on May 8 thru May 12, 1979.

Written system description (as built) including:

- List of Support Systems for Auxiliary Feed System Operation (Both Electric and Steam)
- Water Supplies for AFWS (primary and backup)

Current operating procedures and test and maintenance requirements including:

- All LCO's for AFWS, main FW system and related support systems.
- Listing of operator actions (local and/or control room) and timing requirements for such actions.
- Procedures for reinitiating main feedwater flow.

As Built P&IDs with symbol keys including condensate and steam side

Ledgible Equipment layouts drawings including:

- Isometrics, if available
- Identification of inhibits preventing accessibility to AFWS components and related electrical equipment

Relevant control systems description including:

- Schematic or logic control diagrams
- Listing of actuation signals/logic and control
  - MSIS logic for isolating AFWS, if installed
  - electric power dependences
  - All "readouts" available in control room for AFWS operation

AC & DC Power

- One line diagrams (normal and emergency power supplies)
- Divisional designation e.g., Train A, Train B, requirements on all AFWS components and support systems
- List of normal valve states and loss-of-actuation power failure position

Operating Experience, including

- Number of main feedwater interruptions per year experienced to date for each unit
- Number of demands on AFWS per year to date (test and actual) for each unit
- Summary of AFWS malfunctions, problems, failures

Provide Available reliability analyses

Steam Generator dry-out times (assuming loss of all feedwater flow, with 100% initial power, with Reactor trip, no line breaks)

System design bases including:

- Seismic and environmental qualification
- Code and Quality, QA

Provide written responses to the following set of questions by 5/8/79

Describe backup systems available (to auxiliary feedwater) for providing feedwater to steam generators. Discuss actions and time required to make these systems available. Are procedures available? If so, provide.

Provide the following procedures:

- loss of offsite power
- loss of feedwater
- LOCA (small and large)
- Steam Line Break

Provide following information for PORV's:

- Number
- capacity
- setpoints (open and close)
- manufacturer and model
- indications of position
- record of periods isolated (isolation valve shut)
- challenges during life of plant (from plant records)  
including performance of valve, cause of challenge.
- experience of two-phase or subcooled discharge of PORVs and safety valves with description of valve performance

Provide indications of PORV isolation valve in the control room.

Provide the following information on ECCS:

- initiation setpoints
- system description
- pump performance characteristics (head curves)

Provide reactor protection system trip setpoints.

Provide information on charging pumps, how they relate to ECCS including:

- number
- flow vs. pressure
- power sources and backup
- water sources
- seismic qualification

List all challenges (and cause) to ECCS as indicated on plant records.

List and discuss all instances during which your plant has undergone natural circulation.

Describe all automatic and manual features which can stop the reactor coolant pumps.

ALABAMA POWER COMPANY

MEETING ON 5/8/79

LIST OF ATTENDEES

Ed Reeves	NRC/DOR
Bill LeFave	NRC/ASB
Stu Asselin	Sandia Labs (For NRC/PAS)
Gordon Edison	NRC/PAS
S. H. Hanauer *	NRC/DSS
P. D. O'Reilly	NRC/DPM
T. N. Epps	Alabama Power Co.
Dan Poole	Alabama Power Co.
J. N. Alvarez	Bechtel Corp.
H. G. Huff	Bechtel Corp.
Richard Gallagher	Bechtel Corp.

\* Part Time

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