(7-77)	LICENSEE EVENT REPORT
	CONTROL BLOCK:
	$ \begin{array}{ c } \hline N & c & e & 2 & 2 & 0 & 0 & 0 & 0 & 0 & 0 & 0 & 0 & 0 & 0 $
	REPORT L 6 0 5 0 - 0 3 2 4 7 0 5 1 9 7 9 8 0 6 1 8 7 9 9 SOURCE 60 61 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 80 9 EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (1)
02	During a review of startup data following a reactor startup, it was noted that
03	[PT 2.6.5, Primary Containment Leak Rate Test on the Airlock, was not completed within]
04	the required 72 hours after reaching condition 3. Condition 3 was reached at 0640 on
05	May 14, 1979 and the leak rate test was not done until May 19, 1979, for a total
06	elapsed time of approximately 133 hours.
0 7	(Technical Specifications 3.6.1.3, 6.9.1.9c)
0 7 8	9
09	SYSTEM CAUSE CAUSE CAUSE CAUSE COMPONENT CODE SUBCODE SUBCODE SUBCODE SUBCODE COMPONENT CODE SUBCODE S
	Image: Description of the point of the p
	ACTION FUTURE EFFECT SHUTDOWN TAKEN ACTION ON PLANT METHOD HOURS 22 ATTACHMENT NPRD-4 PRIME COMP. COMPONENT X (B) Z (19) Z (20) Z (21) 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0
10	PT 2.6.5 was completed satisfactorily on the airlock. During this period of operation,
	an administrative oversight prevented the detection of PT 2.6.5 not having been per-
12	formed. The plant startup procedure GP-1 has been revised such that PT 2.6.5 must be
13	completed before the reactor is taken critical.
14	9
15	ACILITY STATUS STATU
	ELEASED OF RELEASE AMOUNT OF ACTIVITY (35) 2 (33) Z (34) NA LOCATION OF RELEASE (36) 9 PERSONNEL EXPOSURES 44 45 80
17 78	NUMBER TYPE DESCRIPTION (39) 0 0 (37) Z (38) NA 9 PERSONNEL INJURIES 13 2347 297 80
18	NUMBER DESCRIPTION (41) 0 0 0 0 NA 9 11 12 80
19 7 8	Loss of or Damage to Facility (43) 7906220400 J 2 10 10
20	PUBLICITY NRC USE ONLY NRC USE ONLY NRC USE ONLY 0 0 0 0 0 0 0 0 0 0 0 0 0
•	9 10 NAME OF PREPARER A. C. Tollison, Jr. PHONE: 919-457-6701