AVERAGE DAILY POWER LEVEL

DOCKET #..... 50-219
UNIT..... O. C. #1
REPORT DATE... December 14, 1979
COMPILED BY... C.M. MCCLAIN
TELEPHONE..... 201-455-8748

MONTH November 1979

DAY	MW	DAY	MW
1.	631.	17.	567.
2.	589.	18.	586.
3.	609.	19.	595.
4.	615.	20.	595.
5.	617.	21.	600.
6.	614.	22.	612.
7.	560.	23.	428.
8.	578.	24.	0.
9.	620.	25.	0.
10.	614.	26.	287.
11.	606.	27.	465.
12.	611.	28.	599.
13.	626.	29.	607.
14.	624.	30.	602.
15.	622.		
16.	615.		

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OPERATING DATA REPORT

OPERATING STATUS

UNIT NAME...OYSTER CREEK

DOCKET NUMBER...50-219

UTILITY DATA PREPARED BY...C.M. MCCLAIN 201-455-8748

REPORTING PERIOD... November 1979

LICENSED THERMAL POWER(MWT)...1930

NAMEPLATE RATING(GROSS MWE)...650

DESIGN ELECTRICAL RATING(NET MWE)...650

MAXIMUM DEPENDABLE CAPACITY(GROSS MWE)...650

MAXIMUN DEPENDABLE CAPACITY(NET MWE)...620

IF CHANGES OCCUR IN CAPACITY RATING SINCE LAST REPORT, GIVE REASON...
NONE

POWER LEVEL TO WHICH RESTRICTED, IF ANY(NET MWE)... NO RESTRICTION

REASON FOR RESTRICTION, IF ANY...
NO RESTRICTION

	MONTH	YEAR	CUMULATIVE
HOURS IN PERIOD	720.0	8016.0	87120.0
HOURS RX CRITICAL	683.2	6874.2	67895.3
RX RESERVE SHUTDOWN HRS.	0.0	0.0	468.2
HRS. GEN ON LINE	663.1	6780.2	66561.5
UT RESERVE SHUTDOWN HRS	0.0	0.0	0.0
GROSS THERMAL ENERGY	1182375.9	12563611.4	112599606.4
GROSS ELEC ENERGY	406050.0	4301430.0	38441435.0
NET ELEC ENERGY	390745.0	4139359.0	37057149.0
UT SERVICE FACTOR	92.1	84.6	76.4
UT AVAILABILITY FACTOR	92.1	84.6	76.4
UT CAPACITY FACTOR MDC	87.5	83.3	70.4
UT CAPACITY FACTOR DER	83.5	79.4	65.4
FORCED OUTAGE FACTOR	7.9	15.4	6.5

THE NEXT SCHEDULED OUTAGE IS TO BEGIN ON JANUARY 5, 1979

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-219
UNIT NAME Oyster Creek #1
DATE December 14, 1979
COMPLETED BY C. M. McClain
TELEPHONE 201-455-8748

REPORT MONTH November 1979

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report #	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
8	112379	F	56.9	G	3	NA	CD	NA	A scram resulted from the inadvertent opening of an isolation condenser return valve during backseating operations. This caused an inrush of cold water into the recirc system resulting in a power spike on the APRM channels.

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance of Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Other (Explain)

4
Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
Exhibit I - Same Source

(9/77)

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OPERATING SUMMARY NOVEMBER 1979

At the beginning of the reporting period the plant was operating at near rated output. On November 6, "A" electromatic relief valve lifted during steady state power operation. The valve was subsequently closed by placing its control switch to "Off" and reactor power was reduced to 1700 MWt to compensate for loss of the high pressure relief function for the affected valve. Following replacement of the pressure switch for the "A" relief valve, the valve control was returned to automatic and plant output was restored to near rated output. On November 23, 1979 a Reactor SCRAM occurred when a technician inadvertently initiated an isolation condenser loop while backseating the loop valves following a surveillance test. The plant was restored to power operation on November 26, following completion of maintenance activities performed subsequent to the SCRAM.

Six Reportable Occurrences were identified during the month.

R.O. #79-37 occurred on November 3, when Core Spray booster pump NZ03B did not start during the regular monthly surveillance test.

R.O. #79-38 occurred on November 5, when Diesel Generator No. 1 failed to complete the starting sequence during a routine surveillance test.

R.O. #79-39 was identified on November 5, when the 100% flow rod block setpoint was found to be greater than the Tech. Spec. limit during performance of the routine surveillance test.

R.O. #79-40 was identified on November 2, when a NRC Audit noted that the Standby Gas Treatment System charcoal filters were not being tested in full compliance with the Technical Specifications.

R.O. #79-41 identified on November 2, when a NRC Audit noted that the Radwaste Building Ventilation Monitoring System was not in service in a manner that could yield accurate effluent data.

R.O. #79-42 occurred on November 6, when the pressure switch for "A" electromatic relief valve failed causing the valve to open during steady state power operation.

Oyster Creek Station #1
Docket No. 50-219

CORRECTIVE ELECTRICAL MAINTENANCE ON QASL ITEMS FOR THE MONTH OF NOVEMBER 1979

Item #	Equipment	Malfunction	Corrective Action
1	DG #1	Output breaker failed to close during surv. test	(1) Adjusted breaker position switch (52HH) (2) Inspected 52HH switch on #2 DG Breaker - found satisfactory
2	Core Spray Booster Pump NZ03B	NZ03B breaker failed to close a second (2nd) time after NZ01B was manually secured during performance of core spray auto start surv. test 610.3.005	(1) Technician noted that NZ03B closing fuse appeared loose - checked fuse & completed surv. satisfactory
3	1-7 Sump Isol. Valve V-24-36	No open indication during testing	(1) Realigned valve position switch
4	Fire Diesel 1-2 Battery B1	Failed to meet "specific gravity" acceptance criteria of 1.25 after an equalizer charge	Replaced battery
5	Air Ejector (gas) Isol Valve V-7-4	Syst I solenoid coil shorted when local fire nozzle initiated inadvertently in August	Replaced solenoid coil during maintenance shutdown
6	"C" Feedwater Pump	Failed to trip when manually securing pump	Replaced shorted trip coil

Oyster Creek Station #1
Docket No. 50-219

CORRECTIVE INSTRUMENT MAINTENANCE ON QASL ITEMS FOR THE MONTH OF NOVEMBER 1979

Item #	Equipment	Malfunction	Corrective Action
1	SRM #21, 22 & 24	Front panel test discrep's (10^5 cps)	Readjusted
2	ARM R010A-3 - Control Room	Initiates alarm prematurely	Performed calibration (as found indication was high)
3	Intake Water Temp Channel	Loss of HP processor & recorder channel indication	Replace & calibrated respective MV/I
4	O_2 Analyzer	Calibr. request to substantiate indication	Replaced defective recorder amplifier & calibrated
5	ERV "A" pressure switch (high pressure trip)	Setpoint drifted down	Replaced pressure switch
6	Rx Press Recorder (NR)	Calibr. request to substantiate indication	1) Minor calibration required 2) Replace drive bushing as PM
7	IRM #16	Front panel test discreps (125%)	Readjusted
8	TIP Detector #1	Detector failed electrically w/ physical damage evident (active and separated from balance of spiral cable upon removal).	Replaced detector
9	SRM's	Front panel discrep's	Readjusted
10	ARM - Old Radwaste Outside	Detector connector loose	Reconnected connector & verified indication

11	"ARM & Vent" Radiation Recorder	Out of step (occasionally)	Lubricated stepping solenoid
12	SRM #21 & 24	Front panel test discreps (10^5 cps)	(1) SRM #21 - replaced meter (2) SRM #24 - adjusted remote signal
13	APRM #5/IRM #15 Recorder	Recorder pen momentarily drives downscale	Cleaned (sprayed) contacts of selector switch
14	N_2 Purge Valve V-23-13	Closes @ 1 psig drywell press - preventing repressurization of DW following vac. bkr. surv.	Recalibrated V-23-13 controller
15	DW/Torus Pressure Recorder	Drive cord loose	Tightened cord pulley
16	"A" ERV Pressure Switch	Found setpoint had drifted down 17 psig from previous surv. calib.	Replaced pressure switch with reconditioned replacement
17	APRM	Front panel test discreps	Required low voltage power supply readjustment
18	LPRM Upscale lites on 4F pnl	Upscale lights dim during period that an upscale on LPRM 12-17A was experienced	Replaced light bulb after finding circuit satisfactory
19	IRM #16	Range 6/7 correlation required	Adjusted R44 in preamp
20	Scram Dump Vol Isolation Solenoid Valve NC-15A	Significant air leak @ upper pilot	Rebuilt NC-15A
21	Rx Level Recorder	Erratic indication	Cleaned slidewire

Oyster Creek Station #1
Docket No. 50-219

CORRECTIVE MECHANICAL MAINTENANCE ON QASL ITEMS FOR THE MONTH OF NOVEMBER 1979

Item #	Equipment	Malfunction	Corrective Action
1	"A" CRD Pump	Oil leak on lower sight glass to oil reservoir	Repaired sight glass
2	CRD Accumulator 38-27	V-111 valve leaking	Installed rebuilt valve
3	CRD Accumulator 06-39	V-111 valve will not close	Installed rebuilt valve
4	Refueling Bridge 119'	Wire rope on bridge mounted aux. hoist badly frayed	Replaced with new wire rope
5	"A" Cleanup pump	Inboard mechanical seal leaking	Replaced with new seal
6	"A" Cleanup pump	Seal water reservoir - sight glass gasket leaking	Gasket surface cleaned, replaced gasket and tightened
7	Fire Protection System	V-9-35 spring on ball check drain valve missing	Replaced spring
8	"A" CRD Filter	Dirty	Installed new filters
9	"B" Cleanup Pump	Outboard mechanical seal leaking	Replaced with new seal

Oyster Creek Station #1
Docket No. 50-219

REFUELING INFORMATION - NOVEMBER 1979

Name of facility: Oyster Creek Station #1

Scheduled date for next refueling shutdown: January 5, 1980

Scheduled date for restart following refueling: March 15, 1980

Will refueling or resumption of operation thereafter require a Technical Specification change or other license amendment?

No Technical Specification change relative to the refueling is anticipated.

Scheduled date(s) for submitting proposed licensing action and supporting information:

1. December 1979 - Cycle independent General Electric fuel design information and safety analysis for future use.

2. No submittal is scheduled for the use of Exxon fuel.

Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

1. General Electric Fuel Assemblies - Fuel design and performance analysis methods have been approved by NRC. New operating procedures, if necessary, will be submitted at a later date.

2. Exxon Fuel Assemblies - No major changes have been made, nor are there are any anticipated.

The number of fuel assemblies (a) in the core - 560
(b) in the spent fuel storage pool - 620

The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

Present: 1,800 Planned: 2,600

The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity:

The Spring 1987 Outage.