

FLORIDA POWER CORPORATION
INTER-OFFICE CORRESPONDENCE

CRYSTAL RIVER NUCLEAR PLANT
(Office - Location)

Subject: NRC Monthly Operating Report

Date: 5/3/79
3-0-1-C2

To: Patsy Baynard

Attention Of:

Attached is the Crystal River Unit #3 MAY, 1979
input to the NRC Monthly Operating Report required by Regulatory Guide 1.16.

Guy Beatty, Jr.
Nuclear Plant Superintendent

2251 120

CC: A.E. Friend
E.C. Simpson
W. A. Stephenson

7906200271

OPERATING DATA REPORT

DOCKET NO. 50-302
 DATE 6/3/79
 COMPLETED BY W. A. Stephenson
 TELEPHONE (904) 795-6486

OPERATING STATUS

1. Unit Name: CRYSTAL RIVER UNIT 3
2. Reporting Period: 5/1/79 - 5/31/79
3. Licensed Thermal Power (MWt): 2452
4. Nameplate Rating (Gross MWe): 890
5. Design Electrical Rating (Net MWe): 825
6. Maximum Dependable Capacity (Gross MWe): 835
7. Maximum Dependable Capacity (Net MWe): 797

Notes

8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons: _____

9. Power Level To Which Restricted, If Any (Net MWe): _____
10. Reasons For Restrictions, If Any: _____

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	<u>744.0</u>	<u>3623.0</u>	<u>19439.0</u>
12. Number Of Hours Reactor Was Critical	<u>0.0</u>	<u>2282.5</u>	<u>12139.7</u>
13. Reactor Reserve Shutdown Hours	<u>0.0</u>	<u>223.4</u>	<u>606.8</u>
14. Hours Generator On-Line	<u>0.0</u>	<u>2254.6</u>	<u>11799.2</u>
15. Unit Reserve Shutdown Hours	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
16. Gross Thermal Energy Generated (MWH)	<u>0.0</u>	<u>5002488.0</u>	<u>25599656.0</u>
17. Gross Electrical Energy Generated (MWH)	<u>0.0</u>	<u>1707369.6</u>	<u>8705215.4</u>
18. Net Electrical Energy Generated (MWH)	<u>0.0</u>	<u>1615921.0</u>	<u>8245785.7</u>
19. Unit Service Factor	<u>0.0</u>	<u>62.2%</u>	<u>60.7%</u>
20. Unit Availability Factor	<u>0.0</u>	<u>62.2%</u>	<u>60.7%</u>
21. Unit Capacity Factor (Using MDC Net)	<u>0.0</u>	<u>56.0%</u>	<u>53.2%</u>
22. Unit Capacity Factor (Using DER Net)	<u>0.0</u>	<u>54.1%</u>	<u>51.4%</u>
23. Unit Forced Outage Rate	<u>0.0</u>	<u>12.9%</u>	<u>32.7%</u>

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): _____

25. If Shut Down At End Of Report Period, Estimated Date of Startup: _____
26. Units In Test Status (Prior to Commercial Operation):

	Forecast	Achieved
INITIAL CRITICALITY	<u> </u>	<u>1/14/77</u>
INITIAL ELECTRICITY	<u> </u>	<u>1/30/77</u>
COMMERCIAL OPERATION	<u> </u>	<u>3/13/77</u>

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-302

UNIT FLCRP-3

DATE 6/3/79

COMPLETED BY W. A. Stephenson

TELEPHONE (904) 795-6486

MONTH MAY, 1979

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	<u>0</u>	17	<u>0</u>
2	<u>0</u>	18	<u>0</u>
3	<u>0</u>	19	<u>0</u>
4	<u>0</u>	20	<u>0</u>
5	<u>0</u>	21	<u>0</u>
6	<u>0</u>	22	<u>0</u>
7	<u>0</u>	23	<u>0</u>
8	<u>0</u>	24	<u>0</u>
9	<u>0</u>	25	<u>0</u>
10	<u>0</u>	26	<u>0</u>
11	<u>0</u>	27	<u>0</u>
12	<u>0</u>	28	<u>0</u>
13	<u>0</u>	29	<u>0</u>
14	<u>0</u>	30	<u>0</u>
15	<u>0</u>	31	<u>0</u>
16	<u>0</u>		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

(9/77)

2251 122

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-302
 UNIT NAME FLCRP-3
 DATE 6/3/79
 COMPLETED BY W. A. Stephenson
 TELEPHONE (904) 795-6486

REPORT MONTH MAY, 1979

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
79-11	790423	S	744	C	1	-----	ZZ	ZZZZZZ	

¹ F: Forced
S: Scheduled

² Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

³ Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Other (Explain)

⁴ Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

⁵ Exhibit I - Same Source

(9/77)

2251 123

MONTHLY STATUS REPORT REFUELING INFORMATION REQUEST

1. Name of Facility: Crystal River Unit 3
2. Scheduled date of next refueling shutdown: April, 1979.
3. Scheduled date for restart following refueling: June, 1979.
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? Yes.
In general, changes to the CR #3 technical specifications will include:
 - a. Moderator Temperature Coefficient (3.1.1.3)
 - b. Control Rod Insertion Limits (3.1.3.6)
 - c. Control Rod Group Assignments (3.1.3.7)
 - d. Axial Imbalance Limits (3.2.1)
 - e. Refueling Boron Concentration (3.9.1)

These specifications will be reviewed and changed as necessary based on the reactivity of the second cycle as compared to that of the first cycle.

5. Scheduled date(s) for submitting proposed licensing action and supporting information: February, 1979.
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, or new operating procedures.

Florida Power Corporation is presently discussing with the NRC staff our intent to request that the power level of CR #3 be raised from the present level of 2452 MW (t) to the ultimate core power level of 2544 MW (t) as described in the CR #3 FSAR. FPC submitted on February 28, 1979 our reload report justifying Cycle 2 operation of CR #3 at 2544 MW (t). On May 25, 1979, FPC modified its Cycle 2 reload report justifying continued operation at 2452 MW (t). It is our intent to continue our discussions with the NRC in order to obtain the power upgrade at a later date.

7. The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool.
 - a) 177 assemblies
 - b) 4 assemblies
8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies.
 - a. Present storage capacity - Pool A - 120 plus 8 failed fuel assemblies
Pool B - 120 plus 8 failed fuel assemblies

8. (Continued)

- b. Filed request on January 9, 1978 with NRC concerning expansion of Pool A from 120 to 544 assemblies plus 6 failed fuel assemblies and expansion of Pool B from 120 to 609 assemblies. Expansion of Pool A is to occur after the refueling in April, 1979. The Pool B expansion will occur at a later refueling outage (approximately 1986).

Additional detailed design information concerning our fuel pool expansion was submitted to the Commission on March 3, March 22, 1978, January 18, 1979, and March 16, 1979.

9. The projected date of the last refueling that can be discharged to the spent fuel assuming the present licensed capacity. 1981-1982.

2251 125