



June 22, 1979

Mr. Harold R. Denton, Director
Office of Nuclear Reactor Regulation
Attention: Mr. O. D. Parr, Chief
Light Water Reactors Branch No. 3
Division of Project Management
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Serial No. 459/060179
PSE&C/JMD:mac:wang

Docket No. 50-339

Dear Mr. Denton:

We have reviewed your letter dated June 1, 1979 concerning, "Instrument Qualification" and we have the following additional information in response to your letter.

The justification for use of the Barton Transmitters at North Anna was presented as an attachment to our letter of August 22, 1978, Serial No. 424. However, additional justification has been provided since that submittal.

In a letter sent to Mr. Harold R. Denton, Serial No. 541, September 29, 1978, we submitted a report entitled "Westinghouse Reactor Protection System/Engineered Safety Features Actuation System Setpoint Methodology." That report contains a table which shows instrument setpoints and allowed deviations. You will note that an allowance of +14% has been made for the Barton 763 Narrow Range Pressurizer Pressure to account for the inaccuracies which were seen during LOCA testing. In a follow-up letter, Serial No. 565, October 13, 1978, we indicated that the trip setpoints at North Anna had been revised based on the new methodology, therefore the environmental affects of the LOCA testing have been accounted for by a setpoint change.

Also, additional testing performed by Westinghouse and reported in NS-TMA-1059, dated September 29, 1978, demonstrates that inaccuracies would be within specification for radiation doses and environments characteristic of design basis events. In the NRC evaluation letter (dated May 15, 1978) of NS-TMA-1950 the Staff did request Westinghouse to perform additional confirmatory testing on production Lot 1 Barton transmitters to demonstrate their long-term acceptability. Westinghouse will be completing an additional test program on the Barton Lot 1 transmitters to provide the additional justification requested by the NRC. Preliminary results are expected to be available by the requested date of September, 1979. Westinghouse states that the details of this additional test program have not yet been finalized, but will be submitted to the NRC within the stipulated period.

The safety evaluation contained in Attachment 2 to Vepco, letter Serial No. 424, states that under a steam line rupture transient condition, reactor vessel integrity is assured far in excess of one effective full power year based on a fracture mechanics analysis performed by Westinghouse.

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