



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

May 16, 1979

Docket Nos. 50-266  
and 50-301

LICENSEE: Northern States Power Company (NSP)  
FACILITY: Prairie Island Nuclear Generating Plant, Unit Nos. 1 and 2  
SUBJECT: SUMMARY OF MEETING HELD ON MAY 12, 1979 TO DISCUSS THE  
AUXILIARY FEEDWATER SYSTEM AND RELATED AREA

On May 12, 1979 representatives of NSP and the Regulatory Staff met to discuss the Auxiliary Feedwater System (AFS) and related areas. The purpose of the meeting was to assemble information for an NRC report. The areas discussed are listed in attachments 1 and 2. The attendees are listed in attachment 3.

The meeting began at 8:00 am with a discussion of the material listed in attachment 1. The information in attachment 2 has been submitted by telecopy on May 11, 1979 and was supplemented on May 12, 1979. The meeting was closed after the licensee reviewed the draft material written as a result of the discussion. All questions in the attachments were answered.

A handwritten signature in cursive script that reads "Marshall Grotenhuis".

Marshall Grotenhuis, Project Manager  
Operating Reactors Branch #1  
Division of Operating Reactors

Attachments:  
As Stated

cc: w/attachments  
See next page

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As part of its on-going review of the Three Mile Island Unit 2 accident, the staff finds that it needs additional information regarding the auxiliary feedwater systems (AFWS). This information as outlined below, is required to evaluate AFWS reliability for Combustion Engineering (CE) and Westinghouse (W) designed pressurized water reactors. The requested information is in addition to that requested in the IE Bulletins, and should be brought to the meeting scheduled with the staff on May 8 thru May 12, 1979.

Written system description (as built) including:

- List of Support Systems for Auxiliary Feed System Operation (Both Electric and Steam)
- Water Supplies for AFWS (primary and backup)

Current operating procedures and test and maintenance requirements including:

- All LCO's for AFWS, main FW system and related support systems.
- Listing of operator actions (local and/or control room) and timing requirements for such actions.
- Procedures for reinitiating main feedwater flow.

As Built P&IDs with symbol keys including condensate and steam side

Ledgible Equipment layouts drawings including:

- Isometrics, if available
- Identification of inhibits preventing accessibility to AFWS components and related electrical equipment

Relevant control systems description including:

- Schematic or logic control diagrams
- Listing of actuation signals/logic and control
  - MSIS logic for isolating AFWS, if installed
  - electric power dependences
  - All "readouts" available in control room for AFWS operation

AC & DC Power

- One line diagrams (normal and emergency power supplies)
- Divisional designation e.g., Train A, Train B, requirements on all AFWS components and support systems
- List of normal valve states and loss-of-actuation power failure position

Operating Experience, including

- Number of main feedwater interruptions per year experienced to date for each unit
- Number of demands on AFWS per year to date (test and actual) for each unit
- Summary of AFWS malfunctions, problems, failures

Provide Available reliability analyses

Steam Generator dry-out times (assuming loss of all feedwater flow, with 100% initial power, with Reactor trip, no line breaks)

System design bases including:

- Seismic and environmental qualification
- Code and Quality, QA

Provide written responses to the following set of questions by 5/8/79

Describe backup systems available (to auxiliary feedwater) for providing feedwater to steam generators. Discuss actions and time required to make these systems available. Are procedures available? If so, provide.

Provide the following procedures:

- loss of offsite power
- loss of feedwater
- LOCA (small and large)
- Steam Line Break

Provide following information for PORV's:

- Number
- capacity
- setpoints (open and close)
- manufacturer and model
- indications of position
- record of periods isolated (isolation valve shut)
- challenges during life of plant (from plant records)  
including performance of valve, cause of challenge.
- experience of two-phase or subcooled discharge of PORVs and safety valves with description of valve performance

Provide indications of PORV isolation valve in the control room.

Provide the following information on ECCS:

- initiation setpoints
- system description
- pump performance characteristics (head curves)

Provide reactor protection system trip setpoints.

Provide information on charging pumps, how they relate to ECCS including:

- number
- flow vs. pressure
- power sources and backup
- water sources
- seismic qualification

List all challenges (and cause) to ECCS as indicated on plant records.

List and discuss all instances during which your plant has undergone natural circulation.

Describe all automatic and manual features which can stop the reactor coolant pumps.

ATTACHMENT 3

LIST OF ATTENDEES

- M. Grotenhuis, Nuclear Regulatory Commission
- A. Bennet, Nuclear Regulatory Commission
- M. Cunningham, Nuclear Regulatory Commission
- J. Gonyeau, Northern States Power Company
- A. Smith, Northern States Power Company

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Docket Files

NRC PDR

Local PDR

ORBI Reading

NRR Reading

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