UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION III
799 ROOSEVELT ROAD
GLEN ELLYN, ILLINOIS 60137

MAY 29 1979

Docket No. 50-484

Northern States Power Company
ATTN: Mr. A. V. Dienhart
Vice President - Tyrone
Energy Park
100 North Barstow Street
Eau Claire, WI 54701

Gentlemen:

This Information Notice is provided as an early notification of a possibly significant matter. It is expected that recipients will review the information for possible applicability to their facilities. No specific action or response is requested at this time. If further NRC evaluations so indicate, an IE Circular or Bulletin will be issued to recommend or request specific licensee actions. If you have questions regarding this matter, please contact the Director of this office.

Sincerely,

James G. Keppler
James G. Keppler
Director

Enclosure: IE Information
Notice No. 79-13

cc w/encl:
Central Files
Director, NRR/DPM
Director, NRR/DOR
PDR
Local PDR
NSIC
TIC
Regions I & IV

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U.S. NUCLEAR REGULATORY COMMISSION
OFFICE OF INSPECTION AND ENFORCEMENT

REGION III

May 29, 1979

IE Information Notice No. 79-13

INDICATION OF LOW WATER LEVEL IN THE OYSTER CREEK REACTOR

Summary

A loss of feedwater transient at the Oyster Creek facility on May 2, 1979, resulted in a significant reduction in water inventory within the reactor core shroud area as measured by one set of water level instruments (triple low level), while the remaining level instruments, sensing from the reactor annulus area indicated water levels above any protective feature setpoint (Figure 1). The water level within the core shroud area was reduced below the "triple low level" setpoint of 4-feet, 8-inches above the top of the fuel.

Subsequent analysis by the licensee has determined that the minimum collapsed water level (solid, without steam voids) over the top of the fuel was 1 to 1-1/2 feet.

Coolant sample analyses and offgas release rates do not indicate any fuel damage occurred.

General

Oyster Creek is a non-jet pump BWR with licensed power of 1930 MWt. The plant was first made critical May 3, 1969.

Status Before Transient

Operating at near full power with the main parameters at levels as follows:

1895 MWt power level
79" Yarway (13'4" over top of fuel) reactor water level
1020 psig reactor pressure
7.1×10^6 #/hr feedflow
14.8×10^4 gpm recirculation flow rate (4 pumps)
12 psid core Δp

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Equipment Out of Service (OOS)

"D" recirc pump OOS due to sea
"B" startup transformer OOS fo

DUPLICATE DOCUMENT

Entire document previously entered into system under:

ANO 7906060164

No. of pages: 11

TERA

Southern California Edison Company

P. O. BOX 800

2244 WALNUT GROVE AVENUE

ROSEMEAD, CALIFORNIA 91770

JACK B. MOORE
VICE PRESIDENT

TELEPHONE
213-572-2292

May 1, 1979

U. S. Nuclear Regulatory Commission
Office of Inspection and Enforcement
Region V
Suite 202, Walnut Creek Plaza
1990 North California Boulevard
Walnut Creek, California 94596

Attention: Mr. R. H. Engelken, Director

Docket No. 50-206
San Onofre Unit 1

Dear Sir:

IE Inspection Report
No. 50-206/79-04

The subject IE Inspection Report was forwarded by letter dated April 11, 1979, from Mr. J. L. Crews of your office. Appendix A to that letter was a Notice of Violation concerning failure of the Reactor Operator Retraining Program to review facility design changes. This letter tenders our response to that Notice of Violation.

1. Cause of the violation

Written station procedures provide for documented review of design changes and other documentation by all licensed operators. In the case of Design Changes 78-16, 78-17 and 78-18, either the procedure for the review was not followed or documentation of the review was misplaced after the review.

2. Steps taken to correct the violation

The three design changes have been reviewed by all licensed operators and results of the review documented.

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3. Steps taken to prevent recurrence

The written procedure governing the review has been modified to insure that all required documents are distributed to licensed operators for review in a timely manner and that documented verification of such reviews can be provided.

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The above corrective actions should eliminate the area of concern noted in the subject report.

Should you have any questions concerning this response, please do not hesitate to contact me.

Sincerely,

J. B. Moore

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