AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-346
UNIT Davis-Besse Unit 1
DATE April 6, 1979
COMPLETED BY Erdal C. Caba
TELEPHONE 419-259-5000, Ext. 236

MONTH March, 1979

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	751	17	755
2	752	18	729
3	737	19	756
4	462	20	760
5	754	21	694
6	752	22	487
7	755	23	544
8	754	24	754
9	746	25	748
10	739	26	705
11	749	27	639
12	748	28	635
13	754	29	631
14	756	30	448
15	756	31	0
16	752		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

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OPERATING DATA REPORT

DOCKET NO. 50-346
DATE April 6, 1979
COMPLETED BY Erdal C. Caba
TELEPHONE 419-259-5000, Ext. 236

OPERATING STATUS

1. Unit Name: Davis-Besse Unit 1
2. Reporting Period: March, 1979
3. Licensed Thermal Power (MWt): 2772
4. Nameplate Rating (Gross MWe): 925
5. Design Electrical Rating (Net MWe): 906
6. Maximum Dependable Capacity (Gross MWe): to be determined
7. Maximum Dependable Capacity (Net MWe): to be determined
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

Notes

9. Power Level To Which Restricted, If Any (Net MWe): None
10. Reasons For Restrictions, If Any:

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	744	2160	13,925
12. Number Of Hours Reactor Was Critical	731.1	1,747.4	8,379.2
13. Reactor Reserve Shutdown Hours	12.9	80.7	871
14. Hours Generator On-Line	712.2	1,675.1	7,408.3
15. Unit Reserve Shutdown Hours	0	0	0
16. Gross Thermal Energy Generated (MWH)	1,602,765	3,879,097	14,066,667
17. Gross Electrical Energy Generated (MWH)	534,407	1,293,268	4,677,023
18. Net Electrical Energy Generated (MWH)	503,590	1,212,558	4,254,018
19. Unit Service Factor	95.7	77.6	56.2
20. Unit Availability Factor	95.7	77.6	56.2
21. Unit Capacity Factor (Using MDC Net)	to be determined		
22. Unit Capacity Factor (Using DER Net)	74.7	61.9	38.5
23. Unit Forced Outage Rate	0	4.8	22.4

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

25. If Shut Down At End Of Report Period, Estimated Date of Startup: April 8, 1979
26. Units In Test Status (Prior to Commercial Operation):

	Forecast	Achieved
INITIAL CRITICALITY		
INITIAL ELECTRICITY		
COMMERCIAL OPERATION		

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REFUELING INFORMATION

DATE: March, 1979

1. Name of facility: Davis-Besse Nuclear Power Station Unit 1

2. Scheduled date for next refueling shutdown: March, 1980

3. Scheduled date for restart following refueling: May, 1980

4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? If answer is yes, what, in general, will these be? If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)?

Yes, see attached

5. Scheduled date(s) for submitting proposed licensing action and supporting information. December, 1979

6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.

None

7. The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool.

(a) 177 (b) 0 (zero)

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies.

Present 260 Increase size by 475 (735 total)

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.

Date March, 1980 - May, 1980 (assuming ability to unload the entire core into the spent fuel pool is maintained).

4. The following Technical Specifications (Part A) will require revision:

2.1.1 & 2.1.2 - Reactor Core Safety Limits (and Bases)
2.2.1 - Reactor Protection System Instrumentation Setpoints (and Bases)
3.1.3.6 - Regulating Rod Insertion Limits
3.1.3.7 - Rod Program
3.2.1 - Axial Power Imbalance (and Bases)

The following Technical Specifications (Part A) may also require revision:

3.1.2.8 & 3.1.2.9 - Borated Water Sources (and Bases)
3.2.4 - Quadrant Power Tilt (and Bases)
3.2.5 - DNB Parameters (and Bases)

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH March, 1979

DOCKET NO. 50-346
UNIT NAME Davis-Besse Unit 1
DATE April 6, 1979
COMPLETED BY Charles N. Alm
TELEPHONE 419-259-5000, Ext. 251

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report #	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
7	79 03 14	S	5.5	B	4	NA	HA	INSTRU	Only the turbine-generator was taken off line to perform further repair work on the turbine's electro-hydraulic control system. The checkout performed indicated suspected failure of backup speed control circuitry. The possible defective circuitry boards were replaced.
8	79 03 21	F	0	A	4	NA	CH	INSTRU	The Main Feedwater Pump Turbine 1 had oscillating speed control problems and was shutdown at 1844 hours. The troubleshooting and operational testing revealed no problems and the pump was returned to service at 1700 hours on March 23, 1979.

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Other (Explain)

4
Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
Exhibit I - Same Source

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UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH March, 1979

DOCKET NO. 50-346
UNIT NAME Davis-Besse Unit 1
DATE April 6, 1979
COMPLETED BY Charles N. Alm
TELEPHONE 419-259-5000, Ext. 251

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report #	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
9	79 03 30	S	26.3	B	1	NA	NA	NA	A unit outage was initiated to perform scheduled maintenance work. The major work items include the repair of Main Steam Safety SP17A2, the check-out and repair of control rod drive motor for Group 8, Rod 6, the repair of Makeup Pump 1, and the inspection of condenser tubes for leaks.

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
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G-Operational Error (Explain)
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Other (Explain)

4
Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
Exhibit I - Same Source

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OPERATIONAL SUMMARY FOR MARCH, 1979

Reactor power was limited to 92.91 percent this entire month because the Main Steam Safety Valve SP17A2 was inoperable.

3/1/79 - 3/3/79

The reactor power level was maintained at 86 percent with the generator gross load at 785 ± 10 MWe until 2200 hours on March 3, 1979. A reactor power decrease to 15 percent was then initiated to enable further repair work of the turbine's electro-hydraulic control system with the turbine-generator off line.

Also, recorder instrumentation was installed on the number 3 turbine control valve to troubleshoot for the cause of oscillation problems.

3/4/79

The turbine-generator was taken off line at 0152 hours, and the electro-hydraulic control system checkout revealed that the backup speed control circuitry may have electrical faults. The defective backup speed control circuit boards were replaced, and the turbine-generator was synchronized on line at 0725 hours.

Reactor power was then increased and attained 86 percent of full power at 1125 hours with the generator gross load at 785 ± 10 MWe.

3/5/79 - 3/9/79

The generator gross load was maintained at 785 ± 10 MWe with reactor power at 86 percent of full power.

3/10/79

Reactor power was decreased and maintained at 75 percent from 0100 hours through 0500 hours to isolate the high pressure feedwater heat exchanger train 2 for the inspection of feedwater heat exchanger 2-4. The inspection was made to determine if the problems discovered in heat exchanger 1-4 were also present in 2-4 and to make the same modifications that were made to 2-4 heat exchanger.

Reactor power was increased at 0500 hours and attained 86 percent of full power at 0600 hours with the generator gross load at 780 ± 10 MWe.

3/11/79

The unit power level remained constant with reactor power at 86 percent, and the generator gross load at 780 ± 10 MWe.

3/13/79 - 3/20/79

Reactor power was increased to 87 percent at 1125 hours on March 13, 1979. The unit power level was then maintained constant with reactor power at 87 percent and the generator gross load at 790 ± 10 MWe until March 18, 1979. Reactor power was then

3/13/79 - 3/20/79 Continued

decreased at 0116 hours to 60 percent to perform turbine stop valve testing. The testing was completed, and reactor power had been returned to 87 percent at 0900 hours on March 18, 1979.

3/21/79

An automatic runback of reactor power to 60 percent was initiated at 1835 hours due to low level in Deaerator #2. The low level was caused by Main Feedwater Pump 1 staying at high speed which forced Main Feedwater Pump 2 to low speed. Reactor power was then manually decreased to 55 percent at 1844 hours, and Main Feedwater Pump 1 was shutdown to enable a troubleshooting check-out.

3/21/79 - 3/23/79

Reactor power was maintained at 60 percent from 1945 hours on March 21, 1979, to 1700 hours on March 23, 1979, with the generator gross load at 520 $\pm$ 10 MWe. The control problem on Main Feedwater Pump 1 could not be determined by the troubleshooting. The Main Feedwater Pump 1 was returned to service at 0735 hours on March 23, 1979, with operational testing until 1700 hours. The operational testing revealed no problems, and reactor power was returned to 87 percent.

3/24/79 - 3/26/79

Reactor power was maintained at 87 percent until 1615 hours on March 26, 1979, when a power decrease to below 75 percent was initiated. The power decrease was required to shutdown the Reactor Coolant Pump 1-1 which had low oil level indication.

3/26/79 - 3/31/79

The generator gross output was held at 655 $\pm$ 10 MWe, with reactor power at 73 percent until 1400 hours on March 30, 1979. At this time, unit load was decreased to perform a scheduled maintenance outage. The turbine generator was taken off line at 2142 hours on March 30, 1979, and the reactor was subcritical at 1109 hours on March 31, 1979. The work items performed during the scheduled maintenance outage will be presented in next month's operational summary.

FACILITY CHANGE REQUESTS COMPLETED DURING MARCH, 1979

FCR NO: 77-413

SYSTEM: Emergency Diesel Generator (EDG) Room Ventilation

COMPONENT: EDG Room Supply Fans 1-1 through 1-4 discharge deflector vanes

CHANGE, TEST, OR EXPERIMENT: On March 8, 1979, the physical work was completed for FCR 77-413, which modified the design of the deflector vanes on the discharge of the EDG Room Supply Fans 1-1 through 1-4. The 1/4" studs which held the vanes in place were removed and replaced with 3/8" rods welded along the length of each of the vanes. The change was made with the approval of the unit architect/engineer, Bechtel Corporation.

REASON FOR THE FCR: The vanes originally were held in place by means of 1/4" studs attached to each end of the vane. The vibration of the vanes due to the air flow across them was causing the original studs to break off.

SAFETY EVALUATION: This change, which replaces the 1/4" studs on the EDG room supply fan discharge deflectors with 3/8" diameter rods, will not affect the safety function of the fans. The change will replace a part that is breaking because of vibration, hence, reliability and availability of the fans will be enhanced.

FACILITY CHANGE REQUESTS COMPLETED DURING MARCH, 1979

FCR NO: 78-496

SYSTEM: Final Safety Analysis Report (FSAR)

COMPONENT: Section 14, Test Abstract of TP 800.14, "Turbine/Reactor Trip Test"

CHANGE, TEST, OR EXPERIMENT: On February 22, 1979, Modification M-2697 was written as per FCR 78-496. This modification deleted the requirement for a 100% reactor trip from Test Procedure TP 800.14. The FCR also verified that the 100% turbine trip test need not be performed provided that the 100% load rejection test is successfully completed.

REASON FOR THE FCR: No commitments were made in the FSAR for a 100% reactor trip test. The 100% load rejection test subjects the reactor and integrated control system (ICS) to transient conditions essentially identical to the 100% turbine trip test. In addition, the 100% load rejection test tests the capability of the electro-hydraulic control (EHC) system to control the turbine-generator's speed at 1800 RPM.

SAFETY EVALUATION: The FSAR made no commitment for the performance of a 100% reactor trip test. The test objectives of the 100% turbine trip test are duplicated in the objectives of the 100% load rejection test. As the transient conditions imposed on the reactor and ICS are essentially identical and the test objectives of the 100% turbine trip test are concurrent with those of the 100% load rejection test, no unreviewed safety questions are involved.