

LICENSEE EVENT REPORT

CONTROL BLOCK:

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0	1	N	Y	J	A	F	1	2	0	0	-	0	0	0	0	-	0	0	0	3	4	1	1	1	1	4			5
7	8	LIC'SSEE CODE						14	15	LICENSE NUMBER										25	LICENSE TYPE					30	57	CAT	58

CON'T

REPORT SOURCE: 01 60 61 0 5 0 0 0 3 3 3 7 69 0 1 2 5 7 74 9 8 75 0 2 2 0 7 9 80

DOCKET NUMBER: 60 61 0 5 0 0 0 3 3 3 7 69 0 1 2 5 7 74 9 8 75 0 2 2 0 7 9 80

EVENT DATE: 60 61 0 5 0 0 0 3 3 3 7 69 0 1 2 5 7 74 9 8 75 0 2 2 0 7 9 80

REPORT DATE: 60 61 0 5 0 0 0 3 3 3 7 69 0 1 2 5 7 74 9 8 75 0 2 2 0 7 9 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0	2	Please See Attachment
0	3	
0	4	
0	5	
0	6	
0	7	
0	8	

7 8 9 80

09		SYSTEM CODE		CAUSE CODE		CAUSE SUBCODE		COMPONENT CODE						COMP. SUBCODE		VALVE SUBCODE																																					
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23																																					
7		Z Z		A		A		Z Z Z Z Z Z Z						Z		Z																																					
17		EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE		REPORT TYPE		REVISION NO.		ACTION TAKEN		FUTURE ACTION		EFFECT ON PLANT		SHUTDOWN METHOD		HOURS		ATTACHMENT SUBMITTED		NPRD-4 FORM SUB.		PRIME COMP. SUPPLIER		COMPONENT MANUFACTURER																									
21		22		23		24		25		26		27		28		29		30		31		32		33		34		35		36		37		38		39		40		41		42		43		44		45		46		47	
7		9		008		03		L		0		G		Z		Z		Z		0000		Y		N		Z		Z999																									

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1	0	Please See Attachment
1	1	
1	2	
1	3	
1	4	

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50

FACILITY STATUS (28) E % POWER (29) 0 7 3 OTHER STATUS (30) NA METHOD OF DISCOVERY (31) A DISCOVERY DESCRIPTION (32) Administrative Review

ACTIVITY CONTENT
RELEASED OF RELEASE

1 6 2 33 34

7 8 9 10 11

AMOUNT OF ACTIVITY (35)
NA

44

LOCATION OF RELEASE (36)
NA

45 80

PERSONNEL EXPOSURES									
NUMBER			TYPE	DESCRIPTION					
1	7	0	0	0	(37)	Z	(38)	NA	

PERSONNEL INJURIES		NUMBER		DESCRIPTION	
1	8	0	0	0	NA

7		8		9		10		11		12		13		14		15		16		17		18		19		20		21		22		23		24		25		26		27		28		29		30		31		32		33		34		35		36		37		38		39		40		41		42		43		44		45		46		47		48		49		50		51		52		53		54		55		56		57		58		59		60		61		62		63		64		65		66		67		68		69		70		71		72		73		74		75		76		77		78		79		80		81		82		83		84		85		86		87		88		89		90		91		92		93		94		95		96		97		98		99		100													
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1		2		3		4		5		6		7		8		9		10		11		12		13		14		15		16		17		18		19		20		21		22		23		24		25		26		27		28		29		30		31		32		33		34		35		36		37		38		39		40		41		42		43		44		45		46		47		48		49		50		51		52		53		54		55		56		57		58		59		60		61		62		63		64		65		66		67		68		69		70		71		72		73		74		75		76		77		78																																													

7 8 9 10 80

PUBLICITY

ISSUED DESCRIPTION (45)

2 0 N (44) NA 790226 0394

NRC: USE ONLY

NRC USE ONLY

NAME OF PREPARER W. Verne Childs

PHONE: 315-342-3840

During normal operation, as part of a review of surveillance test data, it was discovered that the Core Performance Surveillance required by Technical Specification, Appendix A, Paragraph 4.1.C, 4.5.H. and 4.5.I had not been performed on January 20, 21, and 22, 1979. The specifications referenced above require that the Minimum Critical Power Ratio (MCPR), Average Planner Linear Heat Generation Rate (APLHGR), and Linear Heat Generation Rate (LHGR) be determined daily during reactor power operation at greater than 25% of rated power.

As noted in related Licensee Event Reports 79-001 and 79-007, Operations Surveillance Test F-ST-40D titled "Daily Surveillance Checks" was being developed and revised to include a check-off of a number of surveillance requirements. This new surveillance test (F-ST-40D) was reviewed by the Plant Operations Review Committee and approved on January 31, 1979 and implemented on February 1, 1979. In addition, the Operations Superintendent has carefully reviewed the data of F-ST-40D and other surveillance requirements to preclude recurrence of this and similar events.