# AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-368
UNIT ANO-2
DATE 02-13-79
COMPLETED BY C. N. Shively
TELEPHONE 501-968-2519

MONTH January

| DAY | AVERAGE DAILY POWER LEVEL (MWe-Net) | DAY | AVERAGE DAILY POWER LEVEL (MWe-Net) |
|---|---|---|---|
| 1 | 87 | 17 | 86 |
| 2 | 27 | 18 | 90 |
| 3 | 0 | 19 | 51 |
| 4 | 3 | 20 | 91 |
| 5 | 56 | 21 | 95 |
| 6 | 11 | 22 | 91 |
| 7 | 5 | 23 | 92 |
| 8 | 82 | 24 | 93 |
| 9 | 81 | 25 | 93 |
| 10 | 58 | 26 | 92 |
| 11 | 50 | 27 | 93 |
| 12 | 83 | 28 | 93 |
| 13 | 85 | 29 | 92 |
| 14 | 90 | 30 | 93 |
| 15 | 91 | 31 | 41 |
| 16 | 89 | | |

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

(9/77)

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# OPERATING DATA REPORT

DOCKET NO. 50-368
DATE 02-13-79
COMPLETED BY C. N. Shively
TELEPHONE 501-968-2519

## OPERATING STATUS

1. Unit Name: Arkansas Nuclear One - Unit 2
2. Reporting Period: January 1-31, 1979
3. Licensed Thermal Power (MWt): 2815
4. Nameplate Rating (Gross MWe): 959
5. Design Electrical Rating (Net MWe): 912
6. Maximum Dependable Capacity (Gross MWe): NA
7. Maximum Dependable Capacity (Net MWe): NA
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

None

Notes

9. Power Level To Which Restricted, If Any (Net MWe): None
10. Reasons For Restrictions, If Any: NA

| | This Month | Yr.-to-Date | Cumulative |
|---|---|---|---|
| 11. Hours In Reporting Period | 744.0 | 744.0 | 1488.0 |
| 12. Number Of Hours Reactor Was Critical | 679.8 | 679.8 | 1114.7 |
| 13. Reactor Reserve Shutdown Hours | 2.7 | 2.7 | 144.9 |
| 14. Hours Generator On-Line | 602.0 | 602.0 | 654.8 |
| 15. Unit Reserve Shutdown Hours | 1.1 | 1.1 | 1.1 |
| 16. Gross Thermal Energy Generated (MWH) | 321275.0 | 321275.0 | 365853.0 |
| 17. Gross Electrical Energy Generated (MWH) | 70021.0 | 70021.0 | 75588.0 |
| 18. Net Electrical Energy Generated (MWH) | 52357.0 | 52357.0 | 56341.0 |
| 19. Unit Service Factor | | | |
| 20. Unit Availability Factor | | | |
| 21. Unit Capacity Factor (Using MDC Net) | NA Until Commercial Operation | | |
| 22. Unit Capacity Factor (Using DER Net) | | | |
| 23. Unit Forced Outage Rate | | | |

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

NA

25. If Shut Down At End Of Report Period, Estimated Date of Startup: NA
26. Units In Test Status (Prior to Commercial Operation):

| | Forecast | Achieved |
|---|---|---|
| INITIAL CRITICALITY | | 12- 5-78 |
| INITIAL ELECTRICITY | - | 12-26-78 |
| COMMERCIAL OPERATION | 4-5-79 | - |

## UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH January

DOCKET NO. 50-368
UNIT NAME ANO-2
DATE 02-13-79
COMPLETED BY C. N. Snively
TELEPHONE 501-968-2519

| No. | Date | Type[1] | Duration (Hours) | Reason[2] | Method of Shutting Down Reactor[3] | Licensee Event Report # | System Code[4] | Component Code[5] | Cause & Corrective Action to Prevent Recurrence |
|---|---|---|---|---|---|---|---|---|---|
| 79-1 | 790102 | F | 57.7 | A | 1 | NA | HH | Turbin | Main Feedwater Pump Trip |
| 79-2 | 790104 | F | 7.7 | A | 1 | NA | HA | Turbin | Main Turbine High Bearing Vibration |
| 79-3 | 790105 | F | 42.4 | A | 3 | NA | IA | Instru | Irratic CPC Operation |
| 79-4 | 790110 | F | 12.9 | A | 1 | NA | HA | Turbin | Main Turbine High Bearing Vibration |
| 79-5 | 790119 | F | 10.4 | A | 3 | NA | HH | NA | Low SG Level due to Feedwater Pump Trip |
| 79-6 | 790131 | S | 1.1 | B | NA | NA | HA | NA | Turbine Overspeed Testing |
| 79-7 | 790131 | F | 9.8 | A | 3 | NA | HH | NA | High SG Level Received While Placing Feedwater Regulation Valve in Service |

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Other (Explain)

4
Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
Exhibit I - Same Source

(9/77)

REFUELING INFORMATION

DATE: January 1979

1. Name of facility. Arkansas Nuclear One - Unit 2

2. Scheduled date for next refueling shutdown. 03-01-80

3. Scheduled date for restart following refueling. 06-01-80

4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?
If answer is yes, what, in general, will these be?
If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)?

Yes. Description of effects of new core loading.

5. Scheduled date(s) for submitting proposed licensing action and supporting information. 01-01-80

6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.

None

7. The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool. a) 177 b) 0

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies.

present 486 increase size by 0

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.

DATE: March, 1988

NCR MONTHLY OPERATING REPORT
Operating Summary - January, 1979
Unit II

The Unit continued with the 20% reactor power test plateau throughout the month with numerous unplanned trips. None of the outages resulted in excessive downtimes. No significant deficiencies were identified with respect to the Power Escalation Test Program.

There were fifteen occurrences during the month of January. Three of the occurrences were due to a frozen Reactor Makeup Water Tank Level Transmitter. Six occurrences were related to Core Protection Calculator problems requiring plant operation with the CPC's in bypass for short periods of time. Plant Protection System instrumentation failures caused two occurrences. The four remaining occurrences involved the CEA Calculators, Reactor Protection System, Sodium Hydroxide System and the Safety Injection Tanks. In all cases, the requirements of the Technical Specification Action Statements were met.