ENCLOSURE

SAFETY EVALUATION
BY THE PLANT SYSTEMS BRANCH
PROPOSED TECHNICAL SPECIFICATION CHANGE - HIGH PRESSURIZER PRESSURE TRIP SETPOINT
ARKANSAS NUCLEAR ONE, UNIT 2 (ANO-2)
DOCKET NO. 50-368

I. INTRODUCTION

The licensee, Arkansas Power and Light Company, in its submittal of November 27, 1979, requested a change to Unit 2 Technical Specifications (T.S.). A description of the proposed change is provided below.

The high pressurizer pressure trip setpoint, Item 4 of Table 2.2-1 of the Technical Specifications is presently $\leq$ 2345 psia. It is proposed to increase the trip setpoint by 23 psi to $\leq$ 2368 psia.

II. EVALUATION

The increase in the high pressurizer pressure trip setpoint of 23 psi is to eliminate a dynamic allowance imposed prior to operation. The test data collected at startup has demonstrated an instrument channel response time less than assumed in the safety analysis. Therefore, the dynamic allowance factor is no longer required. The Reactor Safety Branch, in its safety evaluation report, will address this aspect of the licensee's evaluation.

The existing narrow range pressurizer pressure instrument used for this trip has a range of 1500 to 2500 psi the present trip setpoint is $\leq$ 2345 psia with an allowable drift of 8.887 psi (allowable value of $\leq$ 2353.887 psia). The proposed trip setpoint is $\leq$ 2368 with an allowable drift of 8.887 psi (allowable value of 2376.887 psia). The new trip setpoint is well within the range of the narrow range pressurizer pressure instrument and the allowable drift (8.887 psi) for the proposed setpoint is identical to that for the present setpoint. Therefore, it is concluded that the proposed trip setpoint is acceptable.

III. CONCLUSION

Based on our review of the licensee's submittal, we conclude that the proposed change to the technical specification - high pressurizer pressure trip setpoint of $\leq$ 2368 psia and allowable value of $\leq$ 2376.887 psia to be acceptable. The transient analysis aspects of this proposed change to the T.S. will be addressed in the Reactor Safety Branch's safety evaluation.