NRC FORM 366 (7-77)

U. S. NUCLEAR REGULATORY COMMISSION

EXHIBIT A

LICENSEE EVENT REPORT

CONTROL BLOCK: (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01 LICENSEE CODE: F L C R P 3 (2) LICENSE NUMBER: 0 0 - 0 0 0 0 0 - 0 0 (3) LICENSE TYPE: 4 1 1 1 1 (4) CAT: - - (5)

CON'T 01 REPORT SOURCE: L (6) DOCKET NUMBER: 0 5 0 - 0 3 0 2 (7) EVENT DATE: 1 2 2 1 7 9 (8) REPORT DATE: 0 1 1 1 8 0 (9)

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

02 At 1630 following a reactor trip, Chem/Rad sampling revealed that the
03 Reactor Coolant Dose Equivalent I-131 exceeded 1.0 microcuries per gram contrary
04 contrary to Tech Spec 3.4.8. Redundancy NA. No safety hazard as reactor coolant
05 Reactor Coolant System purification reduced DEI-131 to acceptable levels.
06 The DEI-131 was reduced to .917 micorcuries per gram at 1930 on 22 December
07 1979. This is the thirteenth (13) occurrence reported under this specification.
08

09 SYSTEM CODE: C G (11) CAUSE CODE: X (12) CAUSE SUBCODE: Z (13) COMPONENT CODE: Z Z Z Z Z Z (14) COMP. SUBCODE: Z (15) VALVE SUBCODE: Z (16)

(17) LER/RO REPORT NUMBER: EVENT YEAR: 7 9 — SEQUENTIAL REPORT NO.: 1 0 9 / OCCURRENCE CODE: 0 3 REPORT TYPE: L — REVISION NO.: 0

ACTION TAKEN: Z (18) FUTURE ACTION: Z (19) EFFECT ON PLANT: Z (20) SHUTDOWN METHOD: Z (21) HOURS (22): 0 0 0 0 ATTACHMENT SUBMITTED: Y (23) NPRD-4 FORM SUB.: N (24) PRIME COMP. SUPPLIER: Z (25) COMPONENT MANUFACTURER: Z 9 9 9

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

10 The cause of this event is attributed to leaking fuel pins and an anticipated
11 Iodine transient following a reactor trip. No corrective action deemed
12 necessary as coolant system purification returned DEI-131 to within acceptable
13 limits.
14

15 FACILITY STATUS: G (28) % POWER: 0 0 0 (29) OTHER STATUS (30): NA METHOD OF DISCOVERY: B (31) DISCOVERY DESCRIPTION (32): Chem/Rad analysis

16 ACTIVITY RELEASED: Z (33) CONTENT OF RELEASE: Z (34) AMOUNT OF ACTIVITY (35): NA LOCATION OF RELEASE (36): NA

PERSONNEL EXPOSURES

17 NUMBER: 0 0 0 (37) TYPE: (38) DESCRIPTION (39): NA

PERSONNEL INJURIES

18 NUMBER: 0 0 0 (40) DESCRIPTION (41): NA

LOSS OF OR DAMAGE TO FACILITY (43)

19 TYPE: Z (42) DESCRIPTION: NA

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PUBLICITY

20 ISSUED: N (44) DESCRIPTION (45): NA NRC USE ONLY

NAME OF PREPARER: K. F. Lancaster KFL PHONE: (904) 795-6486

(SEE ATTACHED SUPPLEMENTARY INFORMATION SHEET)

8001220 372

SUPPLEMENTARY INFORMATION

Report No.: 50-302/79-109/03L-0

Facility: Crystal River Unit #3

Report Date: 11 January 1980

Occurrence Date: 21 December 1979

Identification of Occurrence:

Dose Equivalent I-131 greater than 1.0 microcurie per gram of primary coolant contrary to Tech Spec 3.4.8.

Conditions Prior to Occurrence:

Mode 3 hot standby.

Description of Occurrence:

At 1630 following a reactor trip, chemical analysis revealed that the Dose Equivalent I-131 was 3.44 microcuries per gram of primary coolant. Entered the action statement of Tech Spec 3.4.8. The four (4) hour sampling frequency was initiated. The Dose Equivalent Iodine 131 returned to within acceptable limits at 1930 on 22 December 1979.

Designation of Apparent Cause:

The cause of this event is attributed to leaking fuel pins and an anticipated Iodine transient following a reactor trip.

Analysis of Occurrence:

There was no hazard to the plant or general public as sampling demonstrated decreasing levels of DEI-131. The transient was within the capabilities of the plant purification system.

Corrective Action:

No corrective action deemed necessary as reactor coolant purification returned the DEI-131 to within acceptable limits.

Failure Data:

This is the thirteenth (13th) occurrence reported under this Tech Spec.

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/rc

Reactor Power History of Prior

Forty-eight Hours

Item I

Event Date: 21 December 1979

DATE 12.19.79

HOUR	GMWE (E710) °/°FP	TURB G (T856) BTU/KWH	MWTH (P753) °/°FP	NI (P723) °/°FP	RATIO NI/MT °/°FP	RATIO ME/MT °/°FP
1	95.09	9974	98.33	98.40	1.001	.967
2	96.90	9985	99.02	99.20	1.002	.979
3	97.24	9979	99.31	99.20	.999	.979
4	97.68	9979	99.80	99.70	.999	.979
5	97.45	9983	99.59	99.60	1.000	.978
6	97.54	9980	99.63	99.70	1.001	.979
7	97.52	9981	99.67	99.70	1.000	.978
8	97.63	9983	99.67	99.60	.999	.979
9	97.50	9983	99.63	99.70	1.001	.979
10	97.53	9977	99.63	99.80	1.002	.979
11	97.64	9980	99.76	100.00	1.002	.979
12	97.80	9977	99.88	100.20	1.003	.979
13	97.79	9970	99.80	100.10	1.003	.980
14	97.94	9971	99.96	100.20	1.002	.980
15	97.95	9967	99.96	100.10	1.001	.980
16	97.94	9968	99.96	100.10	1.001	.980
17	98.04	9962	99.96	100.00	1.000	.981
18	97.98	9968	99.96	100.10	1.001	.980
19	98.05	9965	100.04	100.00	1.000	.980
20	98.01	9968	100.00	99.90	.999	.980
21	98.01	9968	100.00	99.80	.998	.980
22	98.00	9969	100.00	99.80	.998	.980
23	98.01	9967	100.00	99.80	.998	.980
24	98.01	9971	100.04	99.70	.997	.980

AVERAGE DAILY GENERATOR GROSS 834.78 MWH(E)
AVERAGE DAILY THERMAL POWER 2445.46 MWH(T)
AVERAGE DAILY TURBINE GROSS HEAT RATE 9974 BTU/KWH
AVERAGE DAILY MWTH POWER 99.733 °/°FP
AVERAGE DAILY NUCLEAR INST. POWER 99.767 °/°FP

RATIO OF NI TO MWTH = 1.000

DATE 12.20.79

HOUR	GMWE (E710) /°FP	TURB G (T856) BTU/KWH	MWTH (P753) °/°FP	NI (P723) °/°FP	RATIO NI/MT °/°FP	RATIO ME/MT °/°FP
1	97.78	9976	99.88	99.80	.999	.979
2	97.79	9976	99.51	99.80	1.003	.983
3	97.59	9966	99.55	99.60	1.001	.980
4	97.60	9974	99.67	99.70	1.000	.979
5	97.86	9966	99.84	99.90	1.001	.980
6	97.88	9965	99.84	100.10	1.003	.980
7	97.89	9966	99.88	100.10	1.002	.980
8	97.85	9974	99.92	100.10	1.002	979
9	97.86	9974	99.92	100.10	1.002	.979
10	97.88	9970	99.92	100.10	1.002	.980
11	97.88	9970	99.92	100.00	1.001	.980
12	97.81	9975	99.88	100.00	1.001	.979
13	97.81	9976	99.88	100.00	1.001	.979
14	97.81	9975	99.88	99.90	1.000	.979
15	97.78	9969	99.96	100.10	1.001	.978
16	97.78	9970	99.96	100.10	1.001	.978
17	97.78	9970	99.96	100.10	1.001	.978
18	97.89	9975	99.96	99.70	.997	.979
19	97.87	9976	99.96	99.70	.997	.979
20	97.88	9972	99.92	99.80	.999	.980
21	97.84	9979	99.96	99.80	.998	.979
22	97.79	9984	99.96	99.80	.998	.978
23	97.84	9979	99.96	99.80	.998	.979
24	97.84	9981	99.96	99.80	.998	.979

AVERAGE DAILY GENERATOR GROSS 836.33 MWH(E)
AVERAGE DAILY THERMAL POWER 2448.96 MWH(T)
AVERAGE DAILY TURBINE GROSS HEAT RATE 9973 BTU/KWH
AVERAGE DAILY MWTH POWER 99.876 °/°FP
AVERAGE DAILY NUCLEAR INST. POWER 99.913 °/°FP

RATIO OF NI TO MWTH = 1.000

DATE 12.21.79

HOUR	GMWE (E710) °/°FP	TURB G (T856) BTU/KWH	MWTH (P753) °/°FP	NI (P723) °/°FP	RATIO NI/MT °/°FP	RATIO ME/MT °/°FP
1	97.89	9975	99.96	99.80	.998	.979
2	97.89	9975	99.96	99.70	.997	.979
3	97.81	9983	99.96	99.70	.997	.979
4	97.78	9984	99.96	99.70	.997	.978
5	97.81	9981	99.96	99.80	.998	.979
6	97.60	9978	99.92	100.10	1.002	.977
7	97.80	9983	99.96	100.10	1.001	.978
8	97.86	9981	99.96	100.10	1.001	.979
9	97.80	9985	99.96	100.10	1.001	.978
10	97.79	9988	100.00	100.00	1.000	.978
11	97.85	9980	99.96	100.00	1.000	.979
12	97.80	9983	99.96	100.20	1.002	.978
13	97.80	9986	100.00	100.20	1.002	.978
14	0.00	0	.00	0.00	0.000	0.000
15	0.00	0	.00	0.00	0.000	0.000
16	0.00	0	.00	0.00	0.000	0.000
17	0.00	0	.00	0.00	0.000	0.000
18	0.00	0	.00	0.00	0.000	0.000
19	0.00	0	.00	0.00	0.000	0.000
20	0.00	0	.00	0.00	0.000	0.000
21	0.00	0	.00	0.00	0.000	0.000
22	0.00	0	.00	0.00	0.000	0.000
23	0.00	0	.00	0.00	0.000	0.000
24	0.00	0	.00	0.00	0.000	0.000

AVERAGE DAILY GENERATOR GROSS	452.97	MWH(E)
AVERAGE DAILY THERMAL POWER	1327.67	MWH(T)
AVERAGE DAILY TURBINE GROSS HEAT RATE	5407	BTU/KWH
AVERAGE DAILY MWTH POWER	54.146	°/°FP
AVERAGE DAILY NUCLEAR INST. POWER	54.146	°/°FP

RATIO OF NI TO MWTH = 1.000

FUEL BURNUP BY CORE REGION

ITEM 2

Event Date: 21 December 1979

Item 2

The burnup was calculated at 104.8 EFPD for the three (3) enrichment regions.

REGION	NUMBER of FA	BURNUP	
B	61	MWD/MTU	18,262
C	60	MWD/MTU	13,698
D	56	MWD/MTU	2750
Ave.		MWD/MTU	11807

Clean-up Flow History

Item 3

Event Date: 21 December 1979

ITEM 3

Cleanup flow history forty-eight (48) hours prior to the first sample in which the limit was exceeded indicated a letdown flowrate of forty-seven (47 gpm.).

History of Degassing Operations for This Report

Item 4

Event Date: 21 December 1979

1835 PZR 12/19/79

1645 PZR 12/20/79

0628 PZR 12/21/79

Time Duration When DEI-131 Exceeded 1.0 µCi/gram and
I-131 Analysis Results

ITEM 5

Event Date: 21 December 1979

ITEM 5

As per Technical Specification 3.4.8, the four (4) hour sampling frequency as depicted on the table below was initiated at 1530 on 21 December 1979 and the Dose Equivalent I-131 was 3.44 microcuries per gram. The four (4) hour sampling frequency was terminated at 2330 on 23 December 1979 when the DEI-131 was determined to be .590 microcuries per gram. The time duration when the specific activity of primary coolant exceeded 1.0 microcuries per gram Dose Equivalent is twenty-four (24) hours.

DATE	TIME	DEI-131
12/21/79	1930	2.95
12/22/79	0000	2.64
12/22/79	0330	2.59
12/22/79	0730	1.95
12/22/79	1130	1.36
12/22/79	1530	1.39
12/22/79	1930	.917
12/22/79	2330	.590
12/23/79	0010	.563