



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
REGION I  
631 PARK AVENUE  
KING OF PRUSSIA, PENNSYLVANIA 19406

Docket Nos. 50-317  
50-318

DEC 13 1979

Baltimore Gas and Electric Company  
ATTN: Mr. A. E. Lundvall, Jr.  
Vice President, Supply  
P. O. Box 1475  
Baltimore, Maryland 21203

Gentlemen:

This Information Notice is provided as an early notification of a possibly significant matter. It is expected that recipients will review the information for possible applicability to their facilities. No specific action or response is requested at this time. If further NRC evaluations so indicate, an IE Circular or Bulletin will be issued to recommend or request specific licensee actions. If you have questions regarding this matter, please contact this office.

Sincerely,

  
Boyce H. Grier  
Director

Enclosures:

1. IE Information Notice No. 79-31
2. List of Recently Issued IE Information Notices

CONTACT: L. E. Tripp  
(215-337-5282)

cc w/encls:

R. M. Douglass, Manager, Quality Assurance  
L. B. Russell, Chief Engineer  
W. Gibson, General Supervisor, Operational QA  
R. C. L. Olson, Senior Engineer  
K. H. Sebra, Principal Engineer

1649 013

7912230

153

ENCLOSURE 1

UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
OFFICE OF INSPECTION AND ENFORCEMENT  
WASHINGTON, D.C. 20555

SSINS: 6870  
Accession No.:  
7910250507

IE Information Notice No. 79-31  
Date: December 13, 1979  
Page 1 of 1

USE OF INCORRECT AMPLIFIED RESPONSE SPECTRA (ARS)

Description of Circumstances:

Recently, the NRC was informed that, during a review of valve weights in conjunction with the as-built review for IE Bulletin 79-14 at North Anna Unit 1, it was discovered that incorrect amplified response spectra (ARS) had been used as input to the analyses of some piping on both units 1 and 2. The reanalyses are being performed by Stone & Webster (S&W), the Architect Engineer for the Virginia Electric & Power Company's facilities. Similarly, Public Service Company of New Hampshire informed the NRC that the wrong ARS were used for the design of some containment supports at their Seabrook units. The Architect Engineer for these plants is United Engineers & Constructors (UE&C).

The NRC is currently pursuing this issue in detail with S&W and UE&C to determine the extent of the problem and the generic implications for other facilities.

This Information Notice is provided as an early notification of a possible significant matter. It is expected that recipients will review the information for possible applicability to their facilities. Specific action has been requested by IE Bulletin 79-14. This bulletin requires that input to seismic analyses of piping, which includes the use of the correct ARS, be verified against the as-built condition of the plant. No specific actions are requested in response to this Information Notice. If NRC evaluations so indicate, further licensee actions may be requested or required. If you have any questions regarding this matter, please contact the Director of the appropriate NRC Regional Office.

No written response to this IE Information Notice is required.

1649 014

ENCLOSURE 2

IE Information Notice No. 79-31  
Date: December 13, 1979  
Page 1 of 1

RECENTLY ISSUED IE INFORMATION NOTICES

Information Notice No.	Subject	Date Issued	Issued to
79-22	Qualification of Control Systems	9/14/79	All Power Reactor Facilities with an OL or CP
79-23	Emergency Diesel Generator Lube Oil Coolers	9/26/79	All Power Reactor Facilities with an OL or CP
79-24	Overpressurization of Containment of a PWR Plant After a Main Steam Line Break	10/1/79	All Power Reactor Facilities with an OL or CP
79-25	Reactor Trips at Turkey Point Unit 3 and 4	10/1/79	All Power Reactor Facilities with an OL or CP
79-26	Breach of Containment Integrity	11/5/79	All Power Reactor Facilities with an OL or CP
79-12A	Attempted Damage to New Fuel Assemblies	11/9/79	All Fuel Facilities, Research Reactors and Power Reactors with an OL or CP
79-27	Steam Generator Tube Ruptures at Two PWR Facilities	11/16/79	All Power Reactor Facilities with an OL or CP
79-28	Overloading of Structural Elements Due to Pipe Support Loads	11/16/79	All Power Reactor Facilities with an OL or CP
79-29	Loss of Nonsafety Related Reactor Coolant System Instrumentation During Operation	11/19/79	All Power Reactor Facilities with an OL or CP
79-30	Reporting of Defects and Noncompliances, 10 CFR Part 21	12/6/79	All Power Reactor Facilities with an OL or CP

1649 015