



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

November 16, 1979

Docket No. 50-245

Mr. W. G. Council, Vice President
Nuclear Engineering and Operations
Northeast Nuclear Energy Company
P. O. Box 270
Hartford, Connecticut 06101

Dear Mr. Council:

By letter dated April 16, 1979, we sent you the list of topics which would not be reviewed for your plant in the Systematic Evaluation Program (SEP). These topics were proposed for deletion from the SEP because they were not applicable to your plant or they were being reviewed as part of an ongoing generic issue outside the SEP. Most of the licensees of plants in the SEP provided comments on these lists. We have reviewed those comments and have conducted further internal review of our original listings. As a result, we have enclosed new listings which supersede the lists transmitted on April 16, 1979.

Enclosure 1 is the list of topics which are being reviewed as part of ongoing generic issues outside of the SEP. The SEP will only monitor the status of these topics. If the topic is completed and implemented during the course of the SEP, a topic assessment will be issued which identifies the licensing action and safety evaluation report (SER) supporting the final disposition of that topic. If an implementation position is developed but no licensing action has been taken, the issue will be considered for implementation on SEP plants on a case-by-case basis. If no implementation position has been developed by the end of the SEP, the topic assessment will merely report the status of the generic effort.

Enclosure 2 is the list of topics that have been deleted from the review of your plant because they are not applicable to your facility or site. No topic assessments will be written for these topics.

Sincerely,

for Thomas V. Wambach
Dennis L. Ziemann, Chief
Operating Reactors Branch #2
Division of Operating Reactors

Enclosures:
As stated

cc w/enclosures:
See next page

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cc w/enclosures:

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MILLSTONE 1SEP TOPICS NOT REQUIRING REVIEW DUE TO
ONGOING GENERIC LICENSING ACTIVITIES

TOPIC III-8-A: LOOSE PARTS MONITORING AND CORE BARREL VIBRATION MONITORING

COMMENTS: Implementation status pending RRRC decision.

TOPIC III-8-D: CORE SUPPORTS AND FUEL INTEGRITY

COMMENTS: This topic is being reviewed generically by DOR under TAP A-2 for PWR's. Future A-2 under development for BWR's.

TOPIC III-9: SUPPORT INTEGRITY

COMMENTS: Generically reviewed under TAP A-12.

TOPIC IV-3: BWR JET PUMP OPERATING INDICATIONS

COMMENTS: This topic is being reviewed generically, by DOR, for all jet pump plants (under TAP B-34).

TOPIC V-1: COMPLIANCE WITH CODES AND STANDARDS

COMMENTS: The Engineering Branch, DOR, is performing a generic review of all plants for compliance with inspection requirements of 10 CFR 50.55a(g).

TOPIC V-4: PIPING AND SAFE END INTEGRITY

COMMENTS: Included in the review for 10 CFR 50.55a.

TOPIC V-13: WATER HAMMER

COMMENTS: Listed as TASK A-1 in NUREG-0371, "TASK ACTION PLANS FOR GENERIC ACTIVITIES", Published date November 1978. Generic letter sent to PWR facilities, requirements not established for BWRs.

TOPIC VI-2-A: PRESSURE - SUPPRESSION BWR CONTAINMENTS

COMMENTS: Listed as TASK A-7 and A-39 in NUREG-0371, "TASK ACTION PLANS FOR GENERIC ACTIVITIES", Published dated November 1978. Applies to MARK I facilities (Dresden 2, Millstone 1, and Oyster Creek). Safety objective of Topic VI-2-A is entirely within the scope of these activities.

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TOPIC VI-2-B: SUBCOMPARTMENT ANALYSIS

COMMENTS: Listed as TASK A-2 in NUREG-0371, "TASK ACTION PLANS FOR GENERIC ACTIVITIES", Published date November 1978. Guidance letters issued to PWR licensees October 1975. The staff is currently developing plans for expanding this activity to resolve this issue for BWR facilities (NUREG-0510). Specific requirements and scope have not been established.

TOPIC VI-6: CONTAINMENT LEAK TESTING

COMMENTS: Reviewed under TAP A-23.

TOPIC VI-7-A-4: CORE SPRAY NOZZLE EFFECTIVENESS

COMMENTS: Listed as TASK A-16 in NUREG-0371, "TASK ACTION PLANS FOR GENERIC ACTIVITIES", Published date November 1978. December 1976 - letter issued to licensee.

TOPIC VI-7-D: LONG TERM COOLING PASSIVE FAILURES

COMMENTS: Generically reviewed and determination as stated in NUREG-0138.

TOPIC VII-1-B: TRIP UNCERTAINTY

COMMENTS: Staff evaluations, issued August 1978, state that review is being pursued independent of SEP to determine impact of instrument error and drift upon safety margins of trip setpoints, and that appropriate actions will be taken to assure that adequate safety margins are maintained.

TOPIC VII-4: FAILURES OF NON-SAFETY SYSTEMS AND SELECTED ESF's

COMMENTS: The scope of this topic is contained in TASK A-17 of NUREG-0371, November 1978. The staff is developing a method of review. Criteria has not been established for implementation.

TOPIC VII-5: INSTRUMENTS FOR MONITORING ACCIDENTS

COMMENTS: TASK A-24 of NUREG-0371, November 1978. Criteria under development.

TOPIC VII-6: FREQUENCY DECAY

COMMENTS: The scope of this topic is completely addressed in TASK A-35 (2.C-10) of NUREG-0371, November 1978.

TOPIC VIII-1-A: EFFECTS OF DEGRADED GRID VOLTAGE

COMMENTS: The scope of this topic is contained in TASK A-35 (C-2, 3, 4, and 5) of NUREG-0371, November 1978. Initial letters sent to licensee August 1976.

TOPIC IX-2: OVERHEAD HANDLING SYSTEMS - CRANES

COMMENTS: Reviewed under TAP A-36.

TOPIC IX-6: FIRE PROTECTION

COMMENTS: This topic is being reviewed outside the SEPB as part of the ongoing licensing review implemented as a result of the Browns Ferry incident.

TOPIC XI-1: APPENDIX I

COMMENTS: This topic is part of an ongoing licensing review for all facilities to implement the limits of the Appendix I to 10 CFR 50.

TOPIC XI-2: RADIOLOGICAL MONITORING SYSTEMS

COMMENTS: Listed in NUREG-0371, November 1978, as TASK B-67, criteria and scope of review under development.

TOPIC XIII-1: CONDUCT OF OPERATIONS

COMMENTS: Generically reviewed by DPM with the exception of "operating experience" which will be provided by I&E.

TOPIC XIII-2: SAFEGUARDS/INDUSTRIAL SECURITY

COMMENTS: Ongoing licensing review to implement the requirements of 10 CFR 73.55.

TOPIC XV-21: SPENT FUEL CASK DROP

COMMENTS: Reviewed under TAP A-36.

TOPIC XV-22: ANTICIPATED TRANSIENTS WITHOUT SCRAM

COMMENTS: TASK A-9 of NUREG-0371, November 1978, criteria for implementation under development.

TOPIC XV-24: LOSS OF ALL AC POWER

COMMENTS: Reviewed under TAP A-44.

TOPIC XVII:

QA FOR OPERATIONS

COMMENTS:

Reviewed by QAB for conformance with 10 CFR Appendix B.

ENCLOSURE 2

MILLSTONE UNIT 1SEP TOPICS NOT APPLICABLE TO MILLSTONE UNIT 1

<u>TOPIC NO.</u>	<u>TITLE</u>	<u>COMMENTS</u>
II-4-E	Dam Integrity	N/A to this site
III-7.A	Inservice Inspection, Including Prestressed Concrete Containments With Either Grouted or UngROUTED Tendons	N/A No Current Criteria
III-7-C	Delamination of Prestressed Concrete Containment	N/A to this containment design
III-10-B	Pump Flywheel Integrity	N/A PWR Safety Topic
V-2	Applicability of Code Cases	N/A at this time. To be reviewed for any future modifications using references to Code Cases.
V-3	Overpressure Protection	Safety Issue N/A to BWR's based on operating experience
V-7	Reactor Coolant Pump Overspeed	N/A PWR Safety Topic
V-8	Steam Generator Integrity	N/A PWR Safety Topic
V-9	Reactor Core Isolation Cooling System	N/A to this facility design
VI-2-C	Ice Condenser Containment	N/A to this facility design
VI-7-A-1	Increased Vessel Head Temperature	N/A PWR Safety Topic
VI-7-A-2	Upper Plenum Injection	N/A PWR Safety Topic
VI-7-B	ESF Switchover From Injection to Recirculation	N/A PWR Safety Topic
VI-7.C.3	PWR Loop Isolation Valve Closure	N/A PWR Safety Topic
VI-7-E	ECCS Sump Design	N/A PWR Safety Topic
VI-7-F	ECCS Accumulator Design	N/A PWR Safety Topic
VI-9-A	MSIV Seal System	N/A to this unit design
VII-7	Acceptability of Swing Bus Design on BWR-4 Plants	N/A to this facility design

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MILLSTONE UNIT 1

(Continued)

<u>TOPIC</u>	<u>TITLE</u>	<u>COMMENTS</u>
4	Boron Addition System (PWR)	N/A PWR Safety Topic
X	Auxiliary Feedwater System	N/A PWR Safety Topic
XV-17	Steam Generator Tube Failure (PWR)	N/A PWR Safety Topic
XV-23	Steam Generator Multiple Tube Failures	N/A PWR Safety Topic
Group I (PWR) DBE	All	N/A PWR DBE Review
Group II DBE	Topic XV-6 (Only) Feedwater System Pipe Break	N/A PWR DBE Review
Group III DBE	All	N/A PWR DBE Review
Group VI DBE	Topic SV-12 (Only), Spectrum of Rod Ejection Accidents	N/A PWR DBE Review
Group X DBE	All	N/A PWR DBE Review