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CONTROL NO: 1308

FILE: INCIDENT REPORT

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METROPOLITAN EDISON COMPANY

POST OFFICE BOX 542 READING, PENNSYLVANIA 19603

TELEPHONE 215 - 929-3601



FEB 2 1975

Director Directorate of Licensing Nuclear Regulatory Commission Washington, D.C. 20555

Dear Sir:

Operating License DPR-50 Docket #50-289

In accordance with Technical Specifications for the Three Mile Island Nuclear Station Unit 1, we are reporting the following abnormal occurrence:

- (1) Report Number: A0-50-289/75-03
- (2a) Report Date: FEB 2 1975
- (2b) Occurrence Date: January 23, 1975
- (3) Facility: Three Mile Island Nuclear Generating Station Unit 1
- (4) Identification of Occurrence:

Title: Loss of Fluid from Hydraulic Shock Suppressor

- Type: An abnormal occurrence as defined by the Technical Specifications, paragraph 1.8d, in that the loss of fluid from a hydraulic shock suppressor threatened to cause an Engineered Safeguards feature or system to be incapible of performing its intended function.
- (5) Conditions Prior to Occurrence:

The reactor was at hot standby with major plant parameters as follows:

Fower: Core: 1%

Elec: 0 MW (Gross) RC Flow: 140 x 10⁶ lb/hr RC Pressure: 2155 psig

1482 198

RC Temp.: 536^oF PRZR Level: 200 in. PRZR Temp.: 650^oF

(6) Description of Occurrence:

During a routine inspection of all hydraulic shock suppressors (snubbers) inside the Reactor Building's secondary shield, one snubber (MUE-43, located on the C high pressure injection line) was found to be inoperative due to a loss of fluid. The snubber was replaced with an identical unit that had been rebuilt with ethylenepropylene seals and the new unit was filled with fluid.

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The inspection of snubbers inside the Reactor Building secondary shield was completed and no other inoperable snubbers were found.

(7) Designation of Apparent Cause of Occurrence:

Subsequer: inspection of the defective unit revealed that the reservoir end caps were loose. There was some evidence, although inconclusive, of leakage at the cylinder end caps. No indication of seal deterioration was observed. The cause of the occurrence, therefore, was material.

(8) Analysis of Occurrence:

The worst possible consequence of the loss of fluid from MUE-43 would take place during a seismic event. Excessive vibration could cause the overstress and possible fracture of the C high pressure injection line; however, since the high pressure injection lines are very conservatively designed, such a fracture is considered unlikely under any circumstances. If such a break did occur, a check valve at the point where this line connects to the Reactor Coolant piping loop B cold leg downstream of the C Reactor Coolant Pump would prevent loss of the primary coolant. During an emergency safeguards actuation, the control room operator would notice a high flow rate in the C high pressure injection line and a rising level in the Reactor Building Sump. He would then isolate the C high pressure injection line. In any case, . a make-up pumps would supply an adequate flow to the other three high pressure injection lines to fulfill their design function.

(9) Corrective Action:

Immediate corrective action was taken as described above to replace the defective snubber.

Long-term preventative action continues with the snubber inspection and seal replacement program as described in the letter from R. C. Arnold to A. Giambusso dated February 14, 1974.

The Plant Operations Review Committee and the Station Superintendent have reviewed and approved the corrective actions.

(10) Failure Data

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Previous Failures:

See Abnormal Occurrence Reports AO 50-289/74-14, 74-20, 74-23, 74-25, and 74-30.

Equipment Identification:

Snubber Assembly Brand - Grinnel Cylinder Type - Lynair Bore - 2.5 in. Stroke - 5 in.

Sincerely,

Arnold Vice President

RCA/RSB/tas

cc: Directorate of Regulatory Operations, Region 1
Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, Pennsylvania 19406

1482 200

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