AVERAGE DAILY UNIT POWER LEVEL

P

000	A DIARONAL	
$(\Omega)(\Omega)(D)$		
MAM	CUMBUNAL	

DOCKET NO.	50-368
UNIT	ANO-2
DATE	June 14,1979
COMPLETED BY	C.N. Shively
TELEPHONE	(501) 968-2519

AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVE (MWe-Net)
0	17	0
0	18	0
0	19	0
0	20	0
0	21	0
0	.,	0
0		0
0	25	0
0		0
0	-5	0
0	27	0
0	28	0
0	29	0
0	30	0
0	21	0

INSTRUCTIONS

. 1

On this format, list the overage daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

(9/77)

7906190273

OPERATING DATA REPORT

DOCKET NO	50-30	58	
DATE	June	14,	1979
COMPLETED BY	C.N.	Shiv	ely
TELEPHONE	(501	968	-2519

OPERATING STATUS

1. Unit Name: Arkansas Nuclear One - Unit 2	Notes
2. Reporting Period: May 1-31, 1979	
3. Licensed Thermal Power (MWt):2815	
4. Nameplate Rating (Gross MWe): 959	
5. Design Electrical Rating (Net MWe): 912	
6. Maximum Dependable Capacity (Gross MWe): NA	
7. Maximum Dependable Capacity (Net MWe): NA	
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7	Since Last Report, Give Reasons:

None

9. Power Level To Which Restricted. If Any (Net MWe): _____None______

10 Reasons For Restrictions. If Any: NA

		This Month	Yrto-Date	Cumulative
11.	Hours In Reporting Period	744.0	3623.0	4367.0
12.	Number Of Hours Reactor Was Critical	0.0	680.9	1115.8
13	Reactor Reserve Shutdown Hours	388.3	1705.3	1847.5
14.	Hours Generator On-Line	0.0	603.1	655.9
15.	Unit Reserve Shutdown Hours	0.0	1.1	1.1
16.	Gross Thermal Energy Generated (MWH)	0.0	321836.	366414
1-	Gross Electrical Energy Generated (MWH)	0.0	70137	75704
18.	Net El ctrical Energy Generated (MWH)	0.0	52357.	56341
19	Unit Service Factor	- 7		
20.	Unit Availability Factor			
21.	Unit Capacity Factor (Using MDC Net)	NA Until (ommercial Oner	tion
22	Unit Capacity Factor (Using DER Net)		Charter at the	
23.	Unit Forced Outage Rate			
24	Churdren C. L. J. J. C. M. H. H.			

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

25 26	If Shut Down At End Of Report Period, Estimated Date of Startup: . Units In Test Status (Prior to Commercial Operation):	Forecast	Achieved
	INITIAL CRITICALITY INITIAL ELECTRICITY COMMERCIAL OPERATION		1 <u>2/05/78</u> 1 <u>2/26/7</u> 8
		2307	223

UNIT SHUTDOWNS AND POWER REDUCTIONS

50-368 DOCKET NO. ANO-2 UNIT NAME June 14, 1979 DATE COMPLETED BY C.N. Shively TELEPHONE (501) 968-2519

REPORT MONTH May

No.	Date	Type ¹	Duration (Hours)	Reason	Method of Shutting Down Reactor3	Licensee Event Report #	Sy stem Code ⁴	Conipolient Code5	Cause & Corrective Action to Prevent Recurrence	
79-8	790422	F	300.1	A	NA	79-032	EE	ENGINE	Emergency Diesel Generator motor failure.	
	790513	F	49.2	11	NA	NA	НВ	VALVEX	Main steam safety valve blowdown testing.	
	790515	F	270.2	A	NA	79-033	СВ	РІРЕХХ	RCP "C" Seal pressure sensing line leak.	
	790526	F	39.0	н	NA	NA	НВ	VALVEX	Main steam safety valve blowdown testing.	
	790528	F	85.5	A	NA	NA	OB	РИМРХХ	RCP "C" seal failure.	I UUU
F Fo S Sci	riced acduled	2 Reaso A-Equ B-Mai C-Ref D-Rej I-Ope F-Adu G-Ope H-Oul	n: aipment Fai intenance or ueling golatory Res trator Traini ministrative trational Lin mer (Explain	lure (E) Test striction ing & L ror (Exp	splain) icense Exan i ² 4ia)	ination	3 Method 1-Manu 2-Manu 3-Auto 4-Other	l: al al Scram. matic Scram. (Explain)	4 Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG- 0161) 5 Exhibit 1 - Same Source	MANUGUNIAL

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1. 1

Name of facility. Arkansas Nuclear One - Unit 2
Scheduled date for next refueling stutdown. 09-01-80
Scheduled date for restart following refueling. 12-01-80
Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? If answer is yes, what, in general, will these be? If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)?
Yes. Description of effects of new core loading.
Scheduled date(s) for submitting proposed licensing action and supporting information. 08-01-80 Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.
NONE
The number of fuel assemblies (a) in the core and (b) in the spent (uel storage pool. a) 177 b) 0
The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool. a) 177 b) 0 The present licensed spent fuel pool storage capacity and the size f any increase in licensed storage capacity that has been requested r is planned, in number of fuel assemblies.
The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool. a) 177 b) 0 The present licensed spent fuel pool storage capacity and the size f any increase in licensed storage capacity that has been requested r is planned, in number of fuel assemblies. resent 486 0
The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool. a) 177 b) 0 The present licensed spent fuel pool storage capacity and the size if any increase in licensed storage capacity that has been requested r is planned, in number of fuel assemblies. resent 486 increase size by 0 the projected date of the last refueling that can be discharged o the spent fuel pool assuming the present licensed capacity.
The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool. a) 177 b) 0 The present licensed spent fuel pool storage capacity and the size if any increase in licensed storage capacity that has been requested r is planned, in number of fuel assemblies. resent 486

NRC MONTHLY OPERATING REPORT

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OPERATING SUMMARY - MAY, 1979

UNIT II

The Emergency Diesel Generator #2 repair continued from the previous month until May 4, when acceptance testing was initiated. The testing of the diesel was successfully completed on May 11, and the emergency diesel generator unit was declared operable.

A Crosby pressure relief valve was obtained and mounted on the Main Steam Header in place of one of the existing main steam relief valves. The unit was heated up to Mode 3 condition on May 13. Relief setpoint and blowdown testing was conducted and based on the results, it was decided to replace all of the original main steam code relief valves with Crosby valves.

On May 14, during relief valve testing, a non-isolatable leak was discovered on the "A" reactor coolant pump upper seal cavity pressure sensing line. Following the completion of the seal sensing line repair and Crosby main steam relief valve installation, the unit was heated up to Mode 3 conditions on May 26.

Testing of the new main steam relief valve setpoint and blowdown was begun; however, leakage from the "A" Reactor Coolant Pump seal forced the unit to a Mode 5 condition. Repairs continued throughout the remainder of the month.

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