NRC FORM 366
(7-77)

LICENSEE EVENT REPORT

CONTROL BLOCK: (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1 | I L Q A D 2 (2) | 0 0 0 - 0 0 0 - 0 0 0 (3) | 4 1 1 1 1 (4) | (5)

LICENSEE CODE | LICENSE NUMBER | LICENSE TYPE | CAT

CON'T

0 1 | REPORT SOURCE L (6) | 0 5 0 0 0 2 6 5 (7) | 0 1 1 0 7 9 (8) | 0 1 2 6 7 9 (9)

DOCKET NUMBER | EVENT DATE | REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 | During a routine inspection of the Unit 2 Reactor Water Clean-up pump rooms, a small

0 3 | leak was found on the 2-1205B clean-up pump suction line. A visual inspection of

0 4 | the leak revealed a 3/4 inch linear indication on the spool piece flange adjacent

0 5 | to the 2-1201-55B clean-up pump suction valve. Safe plant operation was not affected

0 6 | by this occurrence because the leak was located in a section of piping outside the

0 7 | primary containment isolation valves. The volume of leakage was small.

0 8 |

0 9 | SYSTEM CODE C G (11) | CAUSE CODE E (12) | CAUSE SUBCODE C (13) | COMPONENT CODE P I P E X X (14) | COMP. SUBCODE A (15) | VALVE SUBCODE Z (16)

(17) LER/RO REPORT NUMBER: EVENT YEAR 7 9 | — | SEQUENTIAL REPORT NO. 0 0 2 | / | OCCURRENCE CODE 0 3 | REPORT TYPE L | — | REVISION NO. 0

ACTION TAKEN B (18) | FUTURE ACTION Z (19) | EFFECT ON PLANT Z (20) | SHUTDOWN METHOD Z (21) | HOURS (22) 0 0 0 0 | ATTACHMENT SUBMITTED Y (23) | NPRD-4 FORM SUB. N (24) | PRIME COMP. SUPPLIER A (25) | COMPONENT MANUFACTURER Z 9 9 9 (26)

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 | The exact cause of this occurrence is unknown. The immediate corrective action was to

1 1 | isolate the leak. The linear indication was ground out. The area was then penetrant

1 2 | tested to verify that the entire indication had been removed. The ground out area

1 3 | was then filled with weld material and dye-penetrant tested.

1 4 |

1 5 | FACILITY STATUS E (28) | % POWER 0 9 7 (29) | OTHER STATUS (30) NA | METHOD OF DISCOVERY A (31) | DISCOVERY DESCRIPTION (32) Operator Observation

1 6 | ACTIVITY RELEASED Z (33) | CONTENT OF RELEASE Z (34) | AMOUNT OF ACTIVITY (35) NA | LOCATION OF RELEASE (36) NA

1 7 | PERSONNEL EXPOSURES NUMBER 0 0 0 (37) | TYPE Z (38) | DESCRIPTION (39) NA

1 8 | PERSONNEL INJURIES NUMBER 0 0 0 (40) | DESCRIPTION (41) NA

1 9 | LOSS OF OR DAMAGE TO FACILITY TYPE Z (42) | DESCRIPTION (43) NA

2 0 | PUBLICITY ISSUED N (44) | DESCRIPTION (45) NA | NRC USE ONLY

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NAME OF PREPARER Jim Swales PHONE: 309-654-2241, ext. 247

GPO 917-926

I. LER NUMBER: LER/RO 79-02/03L-0

II. LICENSEE NAME: Commonwealth Edison Company
Quad-Cities Nuclear Power Station

III. FACILITY NAME: Unit Two

IV. DOCKET NUMBER: 050-265

V. EVENT DESCRIPTION:

During a routine inspection of the Unit 2 clean-up pump rooms on January 10, 1979, a small leak was found on the 2-1205B cleanup pump suction line. The leak was located between pump suction valve 2-1201-55B, and the cleanup pump suction spool piece. The leak was discovered at 0445 hours on January 10 and the 2-1205B cleanup pump was isolated before 1200 hours the same day. A visual inspection of the leak revealed a 3/4 inch linear surface indication on the spool piece flange adjacent to the 2-1201-55B valve. The indication was not located in a weld. The last occurrence of this type was reported in LER/RO 78-23/03L-0.

VI. PROBABLE CONSEQUENCES OF THE OCCURRENCE:

The leak was located in a section of cleanup piping beyond the Primary Containment Isolation Valves, MO-2-1201-2 and MO-2-1201-5; therefore, safe plant operation was not affected by this occurrence. The primary containment isolation function of these valves was not affected. The volume of the leakage was small and was manually isolated by closing valves 2-1201-55B and 2-1201-61B. There were no uncontrolled radioactive releases to the environment or unplanned personnel exposures as a result of this event.

VII. CAUSE:

The exact cause of this occurrence is unknown. The indication occurred in the pump suction spool piece flange just downstream of valve 2-1201-55B. The flange is made of 304 stainless steel, schedule 40S and is located on a 3 inch line.

VIII. CORRECTIVE ACTION:

The immediate corrective action was to isolate the leak and visually examine it. After examination, the linear indication was ground out. It was then dye-penetrant tested to verify that the entire indication had been ground out. The leak was then filled with weld material and penetrant tested once again to verify a proper weld.