U.S. NUCLEAR REGULATORY COMMISSION
OFFICE OF INSPECTION AND ENFORCEMENT

REGION III

Report No. 50-373/78-24; 50-374/78-15

Docket No. 50-373; 50-374 License No. CPPR-99; CPPR-100

Licensee: Commonwealth Edison Company
P. O. Box 767
Chicago, IL 60690

Facility Name: La Salle County Station, Units 1 and 2

Inspection At: La Salle Site, Seneca, IL

Inspection Conducted: October 11-13, 1978

Inspectors: H. S. Phillips 11-29-78

G. F. Maxwell 11-28-78

P. A. Barrett 11/29/78

Approved By: D. W. Hayes, Chief 11/29/78
Projects Section

Inspection Summary

Inspection on October 11-13, 1978 (Reports No. 50-373/78-24; 50-374/78-15)
Areas Inspected: Reactor Buildings Units 1 and 2 and Auxiliary Building housekeeping and status of construction; previously identified unresolved matters and items of noncompliance, Units 1 and 2; containment prestressing Unit 2 (work and records); reactor internals Units 1 and 2 (work); safety related components, Unit 1, (work and records); electrical cables, Unit 1 (work and records), and electrical procedures, Units 1 and 2. The inspection involved a total of 63 inspector-hours onsite by three NRC inspectors.
Results: No items of noncompliance were identified.

DETAILS

Persons Contacted

Principal Licensee Personnel

L. J. Burke, Site Project Superintendent
*B. J. McAndrew, Lead Mechanical Engineer
*E. Wendorf, Field Engineer
M. Lohmann, Field Engineer
*J. E. Steinmetz, Field Engineer
E. Falb, Field Engineer
*R. J. Legner, QA Supervisor
*T. R. Sommerfield, QA Electrical Coordinator
*D. J. Skoza, QA Engineer
*J. R. Kodrick, QA Mechanical Coordinator
*L. Tapella, QA Engineer
*R. Mathews, OAD

H. P. Foley Company

*R. Ouzts, QA Manager
*W. S. Marvin, QC Inspector
*B. Hirst, Director of QA

Morrison Construction Company

R. C. Shultz, Sr, QC Supervisor

Reactor Controls, Inc.

J. Klaus, Site Manager

*Denotes those present at the exit interview.

The inspector also contacted several other licensee and contractor employees while conducting this inspection.

Licensee Action on Previously Identified Items

(Open) Noncompliance (Report No. 373/78-06 and No. 374/78-05): The matter concerning Nelson Stud welding qualification in current/voltage parameter, as-well-as hanger weld joints was reviewed again. This matter will be reviewed during subsequent inspections.

(Closed) Noncompliance (374/78-07-02): Failure to identify nonconforming conditions on unit HPCS pump. The inspector reviewed the nonconformance which was subsequently written and the recommended corrective action.

(Open) Unresolved Matter (Report No. 373/77-11; 374/77-10): Reactor Building flood damage to equipment. This item should be ready to close out by December 1978 according to the licensee.

(Open) Noncompliance (373/78-02-11; 374/78-02-11): E. G. Smith QA Manual not including requirements to assure that: results of inspections are documented and nonconforming conditions are identified, documented and resolved. These two items were identified in IE Reports 50-373/78-02 and 50-374/78-02 Appendix A as items A.1 and A.2. CECo has required that E. G. Smith revise their QA Manual to include requirements for documenting inspection results and provisions for identifying, documenting and resolving nonconforming conditions. This item (373/78-02-11; 374/78-02-11) remains open pending resolution/followup on item A.3 listed in Appendix A of above subject IE reports; items A.1 and A.2 are closed.

(Closed) Noncompliance (373/78-09-01; 374/78-08-01): Measures had not been established to control the in-plant storage of Class 1E instruments. The inspector observed the areas identified in IE Report 50-373/78-09; 50-374/78-08. The instrument racks had been cleaned of debris and enclosed in plywood covers. A weekly surveillance of the instrument racks has been implemented by Morrison Construction Company QC and is being documented on Form PC-108.

(Closed) Noncompliance (373/78-11-01): Battery rack torquing requirements not verified. The inspector reviewed the Torque Verification of Battery Racks record dated July 5, 1978. The record indicated compliance with Gould Manual No. 061976D and Gould letter dated June 14, 1978.

(Closed) Unresolved Item (373/78-11-02; 374/78-09-01): H. P. Foley Company sampling inspection program identified in work instruction HPFCO-WI-400, Rev. 3, paragraph 3.5.1. H. P. Foley Company QC has discontinued the use of the sampling program. One hundred percent of all Class 1E cables are being inspected.

Functional or Program Areas Inspected

Section I

Prepared by H. S. Phillips

Reviewed by D. W. Hayes, Chief
Projects Section

1. Plant Tour

The RIII inspector visually inspected the Auxiliary Building areas which included: Switchgear Room, 710' el; Cable Spreading Room, 749' el; Control Room, 768' el; Heating-Air Conditioning and the Reactor Exhaust Fan, 786' el.

Units 1 and 2 of the Reactor Building areas inspected included the top floor, 832', 820', 807', 786' and 761' and 745' elevations.

The inspector observed work activity, construction progress and general housekeeping.

No items of noncompliance were identified in the above areas.

2. Observation and Record of Containment Prestressing, Activity, Unit 2

Tendons 70-F and 64-F stored on pallets in Unit 2 at the 786' el were visually inspected and were found to be properly protected with a canvas covering, polyethlene sheet and protective grease.

Horizontal tendon 47-E at buttress "F" was being pulled for the button heading. The inspection records reviewed indicated that work had been accomplished per work procedure outlined in Appendix 1 and the procedure was at the work site.

No items of noncompliance were identified.

3. Reactor Internals

a. Unit 1

The inspector observed work activity inside the reactor pressure vessel. The core sparger which will require rework was visually observed. The top guide and bottom of the vessel were inspected. The encore instrumentation guide tube welds had been penetrant tested; however, the

penetrant and developer had not been removed. Reactor Control Inc. personnel agreed to discontinue leaving the penetrant and developer on test pieces for extended periods of time although procedure PE-1, Rev. 5 is not explicit. The procedure does imply timely removal after examination. The inspector has no further questions regarding this matter.

b. Unit 2

The inspector observed work activity inside the vessel. CRD stub tubes were being aligned and were being welded. The tube was being welded per Procedure WP-5. Welder No. 25 qualified August 11, 1978 was observed while welding using submerged arc welding, reverse polarity, 5/32 ENCRFE-3 per SFA 5.11, and operating parameters 70-170 amp/16-24 volts. The inspector observed that the method of monitoring welding amperage was to periodically check welding machine outputs. Further, it was learned that RCI inspection does not record this monitoring of in-process-welding amperage. RCI committed to record such surveillance in the future. The inspector had no further questions regarding this matter.

No items of noncompliance were identified.

4. Safety Related Components Installed By Morrison Construction Company

a. Feedwater System, Unit 1

Drawings M-57 and M-118 were reviewed. Valves 1A21-F065A, 1B21-F065B, 1B21-F023B, 1B21-F010B and 1B21-F011B were selected to visually inspect the workmanship, location and review inspection records. The piping system, which includes valves, are furnished in design lengths and alignment is self verifying. Records consisted of weld record reports on file at Morrison Construction Company. The areas reviewed were satisfactory except containment isolation valve 1B21-F011A was found without a cover with gears exposed. Also, the pinion gear tooth was observed to be severely damaged. The licensee could not determine who removed the cover or when. A nonconformance was initiated before the inspector left the site. Therefore, the inspector has no further questions concerning this matter.

No items of noncompliance were identified.

b. Standby Liquid Control, Unit 1

Drawings M-99 and M-14J were reviewed and valves 1C41-F004A, 1C41-F308, 1C41-F309, 1C41-F006, 1C41-F008, 1C41-F312 and 1C41-F313, located in Unit 1, were visually inspected for placement. Inspection placement records consisted of weld records. The piping extending into the bottom of the vessel was also visually inspected while inside the vessel as a result of inspecting reactor internals described in paragraph 3 above.

c. Reactor Water Cleanup, Unit 1

Drawings M-97 and M-143 were reviewed and valves M01G33-F001 and M01G33-F101, located in Unit 1, were visually inspected for placement. Inspection placement records consisted of weld records.

No items of noncompliance were identified.

5. Safety Related Component Installation Records, Unit 1

The following records of components requiring final alignment were reviewed at the same time nonconformances were reviewed.

Component	Nonconformance Report	Alignment Records
HPCS Pump 1E22-C001	NCR #137	Equipment assembly and installation traveler.
LPCS Pump	#137	Equipment and installation traveler.
	#54	
RHR Pumps 1E12-C002A 1E12-C002C	#137	Equipment and installation traveler.

No items of noncompliance were identified.

Section II

Prepared by G. F. Maxwell and
P. A. Barrett

Reviewed by R. L. Spessard, Chief
Engineering Support
Section 1

1. Observation of Electrical Work Activities

a. The RIII inspectors observed the following installed Class 1E cables:

(1) Power Cables

#1RH010 - Residual Heat Removal Pump Motor A
#1RH016 - Residual Heat Removal Pump Motor B
#1LP010 - Low Pressure Core Spray Pump Motor
#1HP010 - High Pressure Core Spray Pump Motor
#1HP015 - High Pressure Core Spray Cooling Water Pump Motor

(2) Control Cables

#1RH013 - Residual Heat Removal System
#1RH019 - Residual Heat Removal System
#1HP011 - High Pressure Core Spray System
#1LP019 - Low Pressure Core Spray System
#1AP108 - Auxiliary Power System
#1AP098 - Auxiliary Power System
#1LC068 - Leak Control System
#1VP031 - Primary Containment Ventilation System

The cables were installed in accordance with H. P. Foley Company work instruction #HPFCo-WI-400, checklist #HPFCo-041. The raceways were free of debris and sharp edges. The cables were correctly routed and exhibited no separation problems. The cables and raceways were properly identified and undamaged. Kellum grip supports were not located as specified in some cases, but were being controlled on Corrective Action Reports. The inspector observed a continuity test of cable #1AP108 which verified proper identity.

b. The RIII inspectors observed several locations in which nonseismic pipes were installed over Class 1E cables and raceway (i.e., pipe #1MC22A-4" over trays 1337A-1YP, 1337B-1YP, 1337B-1YP, and 1337D-1YP; and drain pipes #ABD-11-6/ABD-11-12 over trays 349A-1BP and 399B-1BC). A letter dated October 12, 1978 from Sargent & Lundy's Project Manager for La Salle Units 1 and 2 to CECo indicates that a general criteria for review of the adequacy of nonsafety related piping systems located above safety related items is being developed. Once the criteria is completed, the La Salle County Station design will be reviewed for adequacy. This matter is unresolved and will be reviewed during a subsequent inspection. (373/78-24-01; 374/78-15-01)

c. The RIII inspector observed the cable and cable tray storage yards. The yards were being maintained in accordance with H. P. Foley Company procedure #QCP-4, Rev. 5. The cable and trays were identified and placed on dunnage.

No items of noncompliance were identified.

2. Review of Electrical Documentation

a. The RIII inspector reviewed the Cable Pulling Inspection Checklists, #HPFCo-041, for cables #1LP010, #1HP010, #1HP015, #1AP108, #1AP098, #1LC068, and #1VP031. The records were identifiable and retrievable. The records identified the inspector, types of observations, the results, the deficiencies, and the acceptability.

b. The RIII inspector reviewed the Electrical Construction Test Procedure #24-Cable Insulation. Paragraph 1.2 of that procedure specifically excludes control cables from insulation resistance testing. Therefore, the inspector inquired as to how compliance with IEEE 336-1971 Section 5.2.1 would be accomplished. The licensee indicated that the tests which are performed, satisfies the requirements of IEEE 336-1971. This matter is unresolved pending further review by IE. (373/78-24-02; 374/78-15-02)

No items of noncompliance were identified.

Unresolved Items

Unresolved items are matters about which more information is required in order to ascertain whether they are acceptable items, items of

noncompliance, or deviations. Unresolved items disclosed during the inspection are discussed in paragraphs 1.b and 2.b of Section II.

Exit Interview

The inspectors met with site staff representatives (denoted in the Persons Contacted paragraph) at the conclusion of the inspection on October 13, 1978. The inspectors summarized the scope and findings of the inspection. The licensee acknowledged the findings.