UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION II
101 MARIETTA ST., N.W., SUITE 3100
ATLANTA, GEORGIA 30303

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JAN 21 1980

In Reply ~~Refer~~ To:
RII:JPO
50-259, 50-260
50-296, 50-518
50-519, 50-520

Tennessee Valley Authority
Attn: H. G. Parris
Manager of Power
500A Chestnut Street Tower II
Chattanooga, Tennessee 37401

Gentlemen:

The enclosed Bulletin No. 80-02 is forwarded to you for action. A written response is required. If you desire additional information regarding this matter, please contact this office.

Sincerely,

James P. O'Reilly
Director

Enclosures:
1. IE Bulletin No. 80-02
2. Listing of IE Bulletins Recently Issued

1930 085

8002060 072

cc w/encl:
H. L. Abercrombie
Plant Superintendent
Post Office Box 2000
Decatur, Alabama 35602

J. F. Cox
400 Commerce Street
W10C131 C-K
Knoxville, Tennessee 37902

D. E. McCloud, Project Engineer
400 Chestnut Street Tower II
Chattanooga, Tennessee 37401

E. G. Beasley
Tennessee Valley Authority
309 Grant's Building
Knoxville, Tennessee 37902

1930 086

SSINS No.: 6820
Accession No.:
7912190653

UNITED STATES
NUCLEAR REGULATORY COMMISSION
OFFICE OF INSPECTION AND ENFORCEMENT
WASHINGTON, D.C. 20555

DUPLICATE

January 21, 1980

IE Bulletin No. 80-02

INADEQUATE QUALITY ASSURANCE FOR NUCLEAR SUPPLIED EQUIPMENT

Description of Circumstances:

The Marvin Engineering Company (MEC) of Inglewood, California was the subject of a special NRC inspection conducted on September 18-21, 1979 to evaluate their overall QA and QC programs which had come under severe criticism by a former Marvin Engineering employee. The special inspection was judged necessary since the Marvin Engineering Company was a known subcontractor and supplier to General Electric of BWR reactor internal feedwater spargers and feedwater thermal sleeves. The results of the NRC's inspection established that serious deficiencies exist in the implementation of the MEC quality assurance program relative to the manufacture of these components. During this inspection, twenty seven deviations from applicable codes, and contractual and regulatory requirements were documented in the areas of material identification and control, process control, welding and nondestructive examination.

Corrective Action

The Marvin Engineering Company has been issued the NRC's detailed inspection report of the Company, and given 30 days to inform the NRC how the deviations discussed in the report will be corrected and preventive action taken.

Action To Be Taken

All BWR licensees and construction permit holders shall supply the following information within 90 days for operating plants and 120 days for plants under construction.

1.a. Have reactor feedwater spargers and thermal sleeves manufactured and/or fabricated by the Marvin Engineering Company been purchased and/or installed in your facility? Since Marvin Engineering is principally a subcontracting Company, determine if your equipment originated with the Marvin Company and was eventually supplied to your facility through another contractor/supplier.

1.b. Provide a description of this equipment to include its purchase date and its design function during both normal and accident conditions.

2. For each piece of identified equipment, provide the performance history associated with its usage. This should include the cause of any failures or malfunctions and the frequency of such events.

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3. Provide information on the suppliers and receiver's QA/QC program in effect at the time of purchase. This information should be discussed in terms of it providing sufficient bases for judging that the integrity of the equipment is sufficient to permit plant operat during normal and accident conditions.

Approved by GAO, B180225 (R0072); clearance expires 7-31-80. Approval was given under a blanket clearance specifically for identified generic problems.

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IE Bulletin No. 80-02
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Enclosure

RECENTLY ISSUED
IE BULLETINS

Bulletin No.	Subject	Date Issued	Issued To
80-02	Inadequate Quality Assurance for Nuclear Supplied Equipment	1/21/80	All BWR power reactor licensees with OL or CP
80-01	Operability of ADS Valve Pneumatic Supply	1/11/80	All BWR power reactor facilities with an OL
79-01B	Environmental Qualification of Class IE Equipment	1/14/80	All power reactor facilities with an OL
79-28	Possible Malfunction of Namco Model EA 180 Limit Switches at Elevated Temperatures	12/7/79	All power reactor facilities with an OL or a CP
79-27	Loss Of Non-Class-1-E Instrumentation and Control Power System Bus During Operation	11/30/79	All power reactor facilities holding OLs and to those nearing licensing
79-26	Boron Loss From BWR Control Blades	11/20/79	All BWR power reactor facilities with an OL
79-25	Failures of Westinghouse BFD Relays In Safety-Related Systems	11/2/79	All power reactor facilities with an OL or CP
79-17 (Rev. 1)	Pipe Cracks In Stagnant Borated Water System At PWR Plants	10/29/79	All PWR's with an OL and for information to other power reactors
79-24	Frozen Lines	9/27/79	All power reactor facilities which have either OLs or CPs and are in the late stage of construction
79-23	Potential Failure of Emergency Diesel Generator Field Exciter Transformer	9/12/79	All Power Reactor Facilities with an Operating License or a construction permit
79-14 (Supplement 2)	Seismic Analyses For As-Built Safety-Related Piping Systems	9/7/79	All Power Reactor Facilities with an OL or a CP

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