LICENSEE EVENT REPORT

CONTROL BLOCK: (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)
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CON'T O 1 SOURCE L O O O O O O O O O
EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10) [0 2 During a refueling shutdown, an engineering investigation and analysis
[0]3 in accordance with IE Bulletin 79-02 indicated that an anchor in the
[0]4 "A" SG auxiliary feedwater line contained a construction deficiency. An]
[0]5 [as-built analysis of the anchor showed that it was capable of performing
old lits support function, but not its restraint function. Failure of this
O 7 support could result in loss of auxiliary feed to the "A" SG, but would
not violate the steam generator pressure boundary.
SYSTEM CAUSE CAUSE SUBCODE COMPONENT CODE SUBCODE SUBC
LER RO EVENT YEAR REPORT NO. 17 REPORT NUMBER 21 22 23 24 26 27 28 29 30 31 31 32 32 34 36 37 40 41 23 42 43 42 43 43 EPPLER REVISION NO. SEQUENTIAL REPORT NO. OCCURRENCE REPORT TYPE NO. OCCURRE
Three of the six bolts in the anchor support plate were improperly
[1] [installed, thus the anchor did not provide a design safety factor of
greater than 2.0. The anchor has been permanently modified such that
design safety factor is now greater than 4.0 and is now fully capable of
performing its required function.
FACILITY STATUS SPOWER OTHER STATUS 30 METHOD OF DISCOVERY DESCRIPTION 32 1 5 H 28 0 0 0 0 29 N/A CONTENT 12 13 44 45 46 DISCOVERY DESCRIPTION 32 80 80
RELEASED OF RELEASE AMOUNT OF ACTIVITY (35) THE RELEASE OF RELEASE (36) N/A PERSONNEL EXPOSURES AMOUNT OF ACTIVITY (35) N/A 80
1 7 0 0 0 37 Z 38 N/A
PERSONNEL INJURIES NUMBER DESCRIPTION 41 1 8 0 0 0 40 N/A
LOSS OF OR DAMAGE TO FACILITY 43 TYPE DESCRIPTION 1 9 Z (42) N/A
7 8 9 10 80
7 912070 433
NAME OF PREPARER C. W. Fay PHONE: 414/277-2811

ATTACHMENT TO LICENSEE EVENT REPORT NO. 79-020/01T-0

Wisconsin Electric Power Company Foint Beach Nuclear Plant Unit 1 Docket No. 50-266

On November 15, 1979, with Unit 1 shut down for refueling, an engineering evaluation of an anchor on the "A" steam generator auxiliary feedwater line inside containment indicated that the anchor did not meet the design safety factor requirements. The analysis, performed on the as-built condition of the anchor, followed discovery of a construction deficiency in that three of the six bolts in the anchor plate were improperly installed. The investigation and analysis was conducted in accordance with the requirements of IE Bulletin 79-02.

The analysis of the as-built condition of the anchor showed that it had a design safety factor of less than 2.0 and, thus, could not be assumed capable of fully providing its restraint function, although it would provide its support function. An engineering judgment, conservatively assuming that the restraint in question was not provided, predicted unacceptable piping stresses (i.e., not within yield).

The potential consequences of the postulated failure of the "A" steam generator auxiliary feedwater line are reduced by the fact that the deficient anchor is upstream of two-line check valves, with one rigid and two spring hangers in between. Also, the anchor is located outside of the steam generator missile shield. These two facts reduce the potential of violating the steam generator pressure boundary. Further, the line to the "A" steam generator can be isolated upstream of the anchor and all auxiliary feedwater flow directed to the "B" steam generator. No problems were identified in the "B" steam generator auxiliary feedwater line anchors.

The anchor has been permanently modified by providing additional support such that it is now fully capable of performing its required function; the safety factor is now greater than 4.0.

Because of differing equipment and piping layouts, the support in question is unique to Unit 1; a comparable support does not exist in Unit 2.

This discovery is reportable per Technical Specification 15.6.9.2.A.9.