

LICENSEE EVENT REPORT

CONTROL BLOCK: _____ (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

1 | G | A | E | I | I | H | 1 | 2 | 0 | 0 | 1 | - | 1 | 0 | 1 | 0 | 1 | 0 | 1 | 0 | 1 | 0 | 1 | 0 | 1 | - | 1 | 0 | 1 | 0 | 3 | 4 | 1 | 1 | 1 | 1 | 1 | 4 | _____ | 5

9 14 15 23 26 30 37 38

LICENSEE CODE LICENSE NUMBER LICENSE TYPE CAT

1 | L | 5 | 0 | 5 | 1 | 0 | 1 | 0 | 1 | 0 | 1 | 3 | 1 | 2 | 1 | 7 | 0 | 1 | 5 | 2 | 3 | 7 | 9 | 8 | 0 | 5 | 1 | 3 | 1 | 0 | 7 | 9 | 9

8 60 61 64 69 74 75 83

REPORT SOURCE DOCKET NUMBER EVENT DATE REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES

2 | During the refueling outage a Bechtel review of Class IE electrical equipment in the |
 3 | drywell, in response to IES-79-01, indicated that the solenoid on reactor sample isol- |
 4 | ation valve B31-F019 did not have a complete temperature and radiation qualification |
 5 | test report document to qualify it to perform under an accident condition. There was |
 6 | no significant effect on plant safety from this event since this valve would have |
 7 | failed in the closed condition and there is a redundant valve to assure that primary |
 8 | containment isolation is met. This is a non-repetitive occurrence. |

9 | C | D | 11 | B | 12 | Z | 13 | V | A | L | V | I | O | P | 14 | F | 15 | E | 16

9 10 11 12 13 14 15 16 17 18 19 20

SYSTEM CODE CAUSE CODE CAUSE SUBCODE COMPONENT CODE COMP. SUBCODE VALVE SUBCODE

17 | 7 | 19 | 0 | 3 | 6 | 0 | 1 | T | 0

21 22 23 24 25 26 27 28 29 30 31

LER PO REPORT NUMBER EVENT YEAR SEQUENTIAL REPORT NO. OCCURRENCE CODE REPORT TYPE REVISION NO.

F | 18 | A | 19 | Z | 20 | Z | 21 | 0 | 1 | 0 | 1 | 0 | Y | 22 | N | 24 | A | 25 | A | 41 | B | 47

23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47

ACTION TAKEN FUTURE ACTION EFFECT ON PLANT SHUTDOWN METHOD HOURS ATTACHMENT SUBMITTED NRC FORM SUB. PRIME COMP. SUPPLIER COMPONENT MANUFACTURER

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS

10 | At the time of installation, the solenoid was not required to have test results for |
 11 | temperature and radiation requirements. The later implemented IES 79-01 required that |
 12 | all equipment in the drywell meet test results for radiation and temperature for acci- |
 13 | dent conditions which the originally installed equipment can not meet. A new ASCO mod- |
 14 | el N8320A185E has been ordered to replace the installed ASCO model 830281 (continued) |

5 | H | 23 | 0 | 1 | 0 | 1 | 0 | N/A | 30 | D | 31 | NSSS | 22

9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50

FACILITY STATUS % POWER OTHER STATUS METHOD OF DISCOVERY DISCOVERY DESCRIPTION

6 | Z | 33 | Z | 34 | N/A | 25 | N/A | 26

9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50

ACTIVITY CONTENT AMOUNT OF ACTIVITY LOCATION OF RELEASE

7 | 0 | 1 | 0 | 1 | 0 | 33 | Z | 34 | N/A | 25

9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50

PERSONNEL EXPOSURES DESCRIPTION

8 | 0 | 1 | 0 | 1 | 0 | 33 | Z | 34 | N/A | 25

9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50

PERSONNEL INJURIES DESCRIPTION

9 | Z | 42 | N/A | 26

9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50

LOSS OF OR DAMAGE TO FACILITY DESCRIPTION

10 | N | 14 | N/A | 27

9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50

PUBLICITY ISSUED DESCRIPTION

2279 251

NAME OF PREPARER: T. V. Greene, Supt. Plt. Eng. Serv. ONE: 912-367-7781

POOR ORIGINAL

7906140551

GPO 81-118

Georgia Power Company
Plant E. I. Hatch
Baxley, Georgia 31513

Cause Description and Corrective Actions (continued)

solenoid and this will be installed prior to startup.

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NARRATIVE FOR LER 79-036-01-T-0

On May 23, 1979, during the 1979 refueling outage a Bechtel Power Corporation review of Class IE electrical equipment in the drywell, in response to IE Bulletin 79-01, indicated that the solenoid on reactor sample isolation valve 831-F019 did not have a complete temperature and radiation qualification test report to qualify it to perform under an accident condition. There was no significant effect on plant safety from this event since this valve would have failed in the closed condition and there is a redundant valve to assure that primary containment isolation is met.

At the time of installation, the solenoid valve was not required to have test results for temperature and radiation requirements. The later implemented IEB 79-01 required that all Class IE electrical equipment in the drywell meet test results for radiation and temperature for accident conditions which the originally installed equipment can not meet. A new ASCO model N8320A185E has been ordered to replace the installed ASCO model 830281 solenoid and this will be installed prior to startup.

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POOR ORIGINAL