



Public Service Electric and Gas Company 80 Park Place Newark, N.J. 07101 Phone 201/430-7000

January 30, 1979

Director of Nuclear Reactor Regulation
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Gentlemen:

HOPE CREEK GENERATING STATION
DOCKET NOS. 50-354 AND 50-355
CODES AND STANDARDS FOR REACTOR
COOLANT PRESSURE BOUNDARY EQUIPMENT

Pursuant to Section 50.55a(a) (2) (i) and (ii) of 10 CFR Part 50, Public Service Electric and Gas Company hereby requests authorization to use certain components in its Hope Creek Generating Station, Units 1 and 2 which were designed to codes which differ from those specified in Subsections (c) through (f) of 10 CFR 50.55a. The specific components for which authorization is requested are set forth in the table attached hereto.

On February 27, 1970, Public Service Electric and Gas Company filed its construction permit application for Hope Creek, Units 1 and 2. Engineering and Construction schedules at that time were based on expected construction permit issuance in 1971. Initial engineering design and procurement activity was based on this forecast. The design, fabrication, and testing of components purchased in this period were based on recognized codes and standards in effect at the time. Since the construction permits were not issued until November 1974, some of the codes used differ from those now listed in 10 CFR 50.55a Subsections (c) through (f). We believe that the costs associated with replacing the components in the table solely for the purpose of changing the design codes would represent a severe hardship without a compensating increase in quality or safety. Furthermore the codes to which the subject equipment and components were designed assure the achievement of an acceptable level of quality and safety.

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Director of Nuclear
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In view of the above, Public Service Electric and Gas Company respectfully requests an authorization from the Commission in accordance with Section 50.55a(a) (2) (i) and (ii) of 10 CFR Part 50 to use the codes referenced for the components listed in the attachment to this letter.

Very truly yours,

A handwritten signature in cursive script, appearing to read "R. L. Mittl".

R. L. Mittl
General Manager - Licensing and Environment
Engineering and Construction Department

HOPE CREEK GENERATING STATION
 UNITS NO. 1 & 2
 DOCKET NOS. 50-354 AND 50-355
 REACTOR COOLANT PRESSURE BOUNDARY
 EQUIPMENT CODE STATUS

<u>Equipment</u>	<u>Purchase Order Date</u>	<u>Code Equipment Was Built to:</u>
Reactor Pressure Vessel-Unit One	May 7, 1970	ASME III, 1968 Edition with Winter 1969 Addenda
Reactor Pressure Vessel-Unit Two	April 30, 1971	ASME III, 1968 Edition with Summer 1970 Addenda
Control Rod Drive Housings	January 5, 1971	ASME III, 1968 Edition with Summer 1970 Addenda
Control Rod Drive	August 18, 1971	ASME III, 1968 Edition with Winter 1970 Addendum
Power Range Monitor Incore Housing	January 5, 1971	ASME III, 1968 Edition with Summer 1970 Addenda
Jet Pump Instrumen- tation Penetration	January 5, 1971	ASME III, 1968 Edition with Summer 1970 Addenda
Main Steamline safety relief valves	January 28, 1971	Nuclear Pump and Valve Code, 1968 Edition with 1970 Addenda
Main Steamline isola- tion valves	October 8, 1969	Nuclear Pump and Valve Code, 1968 Edition
Main Steamline pipe	January 27, 1972	ASME III, 1971 Edition with Summer 1971 Addenda
Main Steamline flow elements	February 5, 1973	ASME III, 1971 Edition with Summer 1972 Addenda
Reactor recirculation system pump	May 7, 1971	Nuclear Pump and Valve Code, 1968 Edition with 1970 Addenda
Reactor recirculation system shut off valves	February 23, 1971	Nuclear Pump and Valve Code, 1968 Edition with 1970 Addenda
Reactor recirculation system, by-pass valve	December 23, 1971	ASME III, 1971 Edition
Reactor recirculation system, pipe	February 25, 1971	ASME III, 1968 Edition with Summer 1970 Addenda