MEETING SUMMARY DISTRIBUTION

ORB#4

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- J. Rosenthal
- E. G. Adensam
- S. H. Weiss

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UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

January 5, 1979

Docket No. 50-302

LICENSEE: FLORIDA POWER CORPORATION (FPC)

FACILITY: CRYSTAL RIVER UNIT NO. 3 (CR-3)

SUBJECT: SUMMARY OF MEETING HELD ON DECEMBER 19, 1978 REGARDING FORTHCOMING APPLICATION FOR POWER INCREASE

A list of attendees is enclosed.

FPC stated that they intend to apply for authorization to increase the licensed core power level at CR-3 from 2452 to 2544 MWt. They desire to effect this increase at the start of Cycle 2 operations, currently scheduled for mid June 1979.

Significant points discussed are summarized below.

- FPC and Babcock & Wilcox (B&W) briefed us on the content of their forthcoming power increase/reload submittal. They pointed out that since the increased power level was used, in most cases, in the FSAR their submittal was not significantly more complex than a normal reload submittal.
- FPC stated that the loss of coolant flow and locked RCP rotor accidents would be reanalyzed. The loss of flow accident analysis would consider the RCP power monitor currently being reviewed by the staff.
- 3. The schedule for submittal and review was discussed. Due to the likelihood of ACRS review we stated that SER completion must be scheduled for May 1, 1979 and therefore the complete application must be available to the staff no later than February 1, 1979. We also stated that this 3 month review period assumed their submittal would be comparable to a reload submittal. FPC indicated possible difficulties with this schedule. They did indicate that meteorological data would be submitted in the near future.
- 4. FPC was informed that they must either support previously determined radiological consequences as applicable or describe and support any

changes. We stated that control room habitability would be reviewed and that ECCS leakage outside containment must be considered in dose determinations for LCCA analysis. We also asked if DBA's analyzed for CR-3 were all inclusive when compared with the current Standard Review Plan. B&W stated that "instrument lines or other small line breaks outside containment" was not analyzed. We stated that we would inform CR-3 if that was necessary.

5. Startup testing was discussed. We informed FPC and B&W that acceptance criteria as well as more stringent "review" criteria should be established for the evaluation of startup test results. These criteria should be provided with their power increase reload submittal. FPC and B&W will review this issue further.

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Christian C. Nelson, Project Manager Operating Reactors Branch #4 Division of Operating Reactors

Enclosure: List of Attendees

Enclosure

CR-3 MEETING 12-19-78

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POWER INCREASE

Name	Organization
C. Nelson	NRC/ORB#4/DOR
J. Burdoin	NRC/PDB/DOR
S. Diab	NRC/PSB/DOR
J. Rosenthal	NRC/RSB/DOR
E. G. Adensam	NRC/EEB/DOR
5. H. Weiss	NRC/RSB/DOR
R. O. Vosburgh	B&W
R. J. Finnin	B&W
B. Simpson	FPC