JERSEY CENTRAL POWER & LIGHT COMPANY
OYSTER CREEK NUCLEAR GENERATING STATION

PROVISIONAL OPERATING
LICENSE NO. DPR-16

Technical Specification
Change Request No. 80
Docket No. 50-219

Applicant submits, by this Technical Specification Change Request No. 80 to the Oyster Creek Nuclear Generating Station Technical Specifications, changes to Specification 3.5.A.9.

JERSEY CENTRAL POWER & LIGHT COMPANY

BY: Ivan R. Finfrock Jr.
Vice President

STATE OF NEW JERSEY
COUNTY OF MORRIS

Sworn and subscribed to before me this 29th day of Jan, 1980.

Phyllis A. Kabis
Notary Public

PHYLLIS A. KABIS
NOTARY PUBLIC OF NEW JERSEY
My Commission Expires Aug. 16, 1984

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UNITED STATES OF AMERICA
NUCLEAR REGULATORY COMMISSION

IN THE MATTER OF	)	
	)	Docket Number 50-219
JERSEY CENTRAL POWER & LIGHT COMPANY	)	

CERTIFICATE OF SERVICE

This is to certify that a copy of Technical Specification Change Request No. 80 for the Oyster Creek Nuclear Generating Station Technical Specifications, filed with the U.S. Nuclear Regulatory Commission on January 29 , 1980, has this 29th day of January, 1980, been served on the Mayor of Lacey Township, Ocean County, New Jersey by deposit in the United States mail addressed as follows:

The Honorable Henry Von Spreckelsen
Mayor of Lacey Township
P. O. Box 475
Forked River, New Jersey 08731

JERSEY CENTRAL POWER & LIGHT COMPANY

BY: Jean R. Fairbanks
Vice President

DATED: January 29 , 1980

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JCP&L GPU

Jersey Central Power & Light Company
Madison Avenue at Punch Bowl Road
Morristown, New Jersey 07960
(201) 455-8200

January 29, 1980

The Honorable Henry Von Spreckelsen
Mayor of Lacey Township
P. O. Box 475
Forked River, New Jersey 08731

Dear Mayor Von Spreckelsen:

Enclosed herewith is one copy of Technical Specification Change Request No. 80 for the Oyster Creek Nuclear Generating Station Technical Specifications.

These documents were filed with the U. S. Nuclear Regulatory Commission on January 29 , 1980.

Very truly yours,

Ivan R. Finfrock, Jr.
Vice President

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Enclosures

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JERSEY CENTRAL POWER & LIGHT COMPANY
OYSTER CREEK NUCLEAR GENERATING STATION
(DOCKET NO. 50-219)
PROVISIONAL OPERATING LICENSE DPR-16

Applicant hereby requests the Commission to change Appendix A to the above captioned license as follows:

1. Section to be changed:

Section 3.5.A.9

2. Extent of Change:

Revise the above section such that a suppression chamber downcomer submergence range of 3.0 to 5.3 feet is acceptable.

3. Change requested:

The requested change is on the attached revised Technical Specification pages 3.5-7 and 3.5-14.

4. Discussion:

This Technical Specification change is a result of the Mark I Containment Long Term Program as presented in General Electric's Report "NEDE-21885-P, Mark I Downcomer Reduced Submergence - Functional Assessment Report." The report was forwarded to the Nuclear Regulatory Commission by General Electric on July 31, 1978.

In order to mitigate the pool swell loads in the torus as a result of a LOCA, the downcomers in the torus shall be truncated such that there is a three foot submergence below the minimum allowable suppression pool water level. General Electric's Report NEDE-21885-P presents the technical basis for the use of three feet minimum downcomer submergence in Mark I plants, via truncation of the existing downcomer length, and indicates that such a modification will not prevent the containment from performing its intended safety function.

containment is required during fuel handling operations and whenever work is being performed on the reactor or its connected systems in the reactor building since their operation could result in inadvertent release of radioactive material.

The standby gas treatment system[6] filters and exhausts the reactor building atmosphere to the stack during secondary containment isolation conditions, with a minimum release of radioactive materials from the reactor building to the environs.

Two separate filter trains are provided each having 100% capacity.[6] If one filter train becomes inoperable, there is no immediate threat to secondary containment and reactor operation may continue while repairs are being made. Since the test interval for this system is one month (Specification 4.5), the time out-of-service allowance of 7 days is based on considerations presented in the Bases in Specification 3.2 for a one-out-of-two system.

References:
(1) FDSAR, Volume I, Section V-1
(2) FDSAR, Volume I, Section V-1.4.1
(3) FDSAR, Volume I, Section V-1.7
(4) Licensing Application, Amendment 11, Question III-25
(5) FDSAR, Volume I, Section V-2
(6) FDSAR, Volume I, Section V-2.4
(7) Licensing Application, Amendment 42
(8) Licensing Application, Amendment 32, Question 3
(9) Robbins, C. H., "Tests on a Full Scale 1/48 Segment of the Humboldt Bay Pressure Suppression Containment," GEAP-3596, November 17, 1960.
(10) Bodega Bay Preliminary Hazards Summary Report, Appendix 1, Docket 50-205, December 28, 1962.
(11) Report H. R. Erickson, Bergen-Paterson to K. R. Goller, NRC, October 7, 1974. Subject: Hydraulic Shock Sway Arrestors.

In conjunction with the Mark I Containment Short Term Program, a plant unique analysis was performed on August 2, 1976, which demonstrated a factor of safety of at least two for the weakest element in the suppression chamber support system. The maintenance of a drywell-suppression chamber differential pressure within the range shown on Figure 3.5-1 with a suppression chamber water level corresponding to a downcomer submergence range of 3.0 to 5.3 feet will assure the integrity of the suppression chamber when subjected to post-LOCA suppression pool hydrodynamic forces.

REQUIRED DRYWELL TO TORUS
DIFFERENTIAL PRESSURE

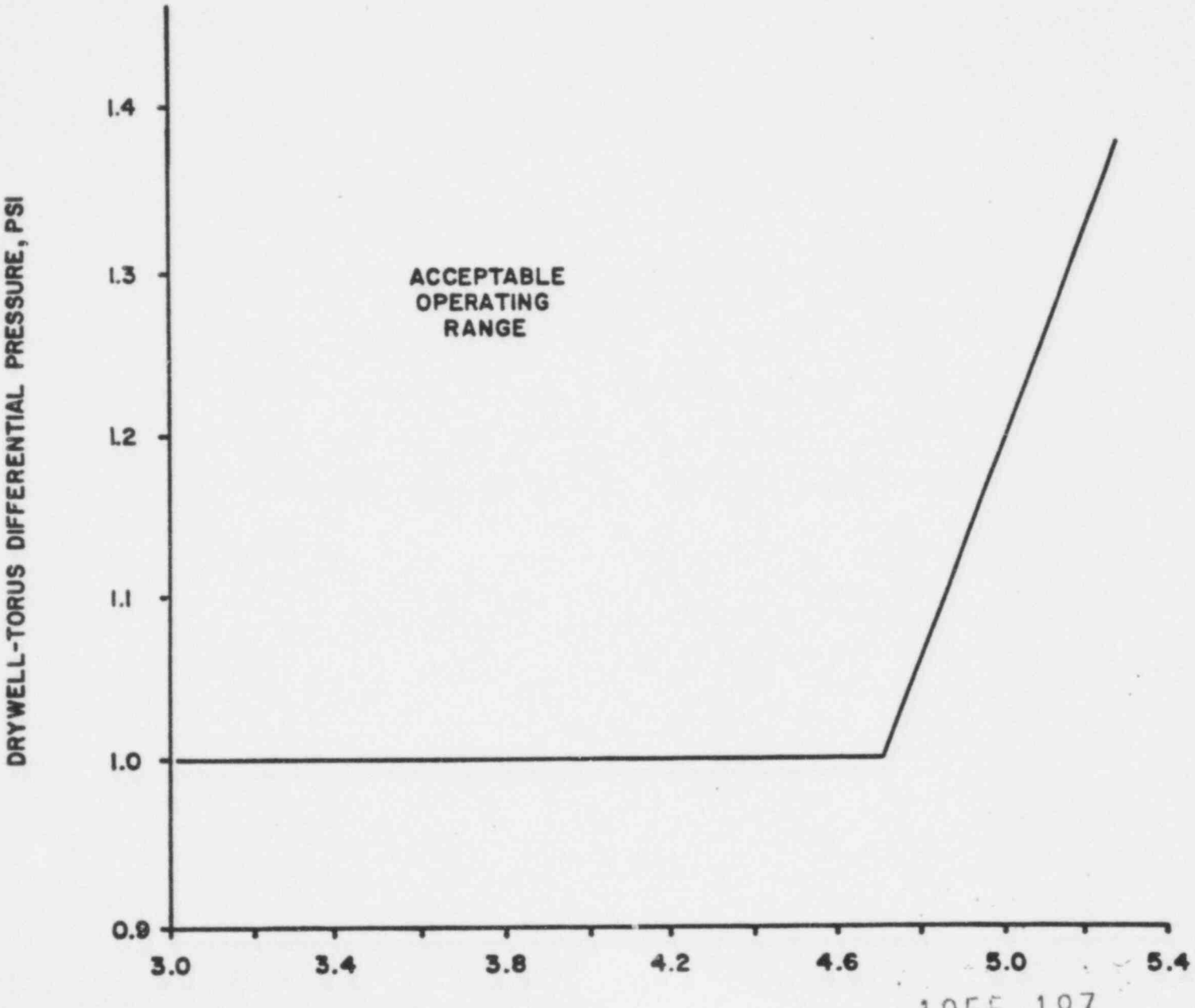

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FIGURE 3.5-1

*The actual acceptable range of downcomer submergence is governed by the Technical Specifications limit on maximum and minimum water volume in the torus (see section 3.5.A.1). This actual acceptable range of downcomer submergence will not encompass the full range of downcomer submergence indicated in the figure above.