

DSB

NJK-79-469

December 24, 1979

J. Keppler, Regional Director  
Office of Inspection and Enforcement  
Region III  
U.S. Nuclear Regulatory Commission  
799 Roosevelt Road  
Glen Ellyn, Illinois 60137

Reference: Quad-Cities Nuclear Power Station  
Docket No. 50-265, DPR-30, Unit 2  
Appendix A, Sections 4.7.A.2.i. and 6.6.B.2.

Enclosed please find Reportable Occurrence Report No. RO 79-27/03L for Quad-Cities Nuclear Power Station.

This report is submitted to you in accordance with the requirements of Technical Specification 6.6.B.1.2, b, as conditions leading to operation in a degraded mode permitted by a limiting condition for operation.

This report identifies six occurrences whereby excessive leakages were measured with respect to primary containment isolation valves during local leak rate testing. The following represents a tabular summary of the leak rate test failures covered in this report.

Main Steam Isolation Valves	A0 2-203-1A	30.0 SCFH
	A0 2-203-2A	47.2 SCFH
	A0 2-203-1B	32.3 SCFH
	A0 2-203-2B	24.2 SCFH
Main Steam Drain Line	M0 2-220-2	26.0 SCFH
RCIC Steam Supply	M0 2-1301-16	76.7 SCFH
Feedwater Check Valve	2-220-62B	405.8 SCFH
Drywell Suppression Chamber Purge Valves	2-1601-21	(Combined leakage
	2-1601-22	for volume)
	2-1601-55	136.1 SCFH
	2-1601-56	
Drywell-Suppression Chamber Vent Valves	2-1601-23	(Combined leakage
	2-1601-24	for volume)
	2-1601-60	27.0 SCFH
	2-1601-61	
	2-1601-62	
	2-1601-63	

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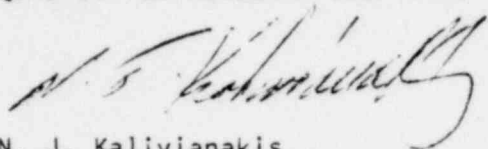
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All valves will be repaired and local leak rate tested prior to unit start-up. A supplemental report will be sent upon completion of all testing and when all corrective actions regarding the above valves have been completed.

Very truly yours,

COMMONWEALTH EDISON COMPANY  
QUAD-CITIES NUCLEAR POWER STATION



N. J. Kalivianakis  
Station Superintendent

NJK/JCV/san

cc: R. F. Janecek