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NORTHERN STATES POWER COMPANY

MINNEAPOLIS, MINNESOTA 55401

December 28, 1979

Director of Nuclear Reactor Regulation
U S Nuclear Regulatory Commission
Washington, DC 20555

PRAIRIE ISLAND NUCLEAR GENERATING PLANT
Docket Nos. 50-282 License Nos. DPR-42
50-306 DPR-60

Reactor Vessel Nozzle Underclad Inspection

In a meeting with representatives of the Westinghouse Electric Corporation and the NRC Staff on November 26, 1979 we outlined our plans for performing inservice inspection of the Prairie Island vessel nozzles. This inspection will be done to confirm our belief that the nozzle cladding process used in the fabrication of the Prairie Island vessels did not give rise to flaws which could impair integrity.

Both outlet nozzles in each unit will be inspected using the underclad ultrasonic examination technique now being developed by Westinghouse. Procedures and equipment for this inspection will be available by the middle of next year. Unit No. 1 will be inspected during the summer 1980 refueling outage and Unit 2 will be inspected during the early 1981 refueling outage. Additional nozzles will be inspected if evidence of significant underclad cracking is found.

In addition to scheduling these inspections, we have authorized Westinghouse to proceed with a reactor vessel safety analysis program for the Prairie Island vessel nozzles. This analysis includes an ASME Code Section III Appendix G analysis per 10 CFR Part 50, Appendix G, accident analyses, and fatigue analyses. These analyses will be available to evaluate the results of the nozzle inservice inspections.

A summary report of the nozzle inservice inspections will be provided to the Commission within 90 days of the completion of each inspection.

Please contact us if you require additional information concerning our plan and schedule for resolving this issue.

L.O. Mayer

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Manager of Nuclear Support Services

LOM/DMM/ak

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