

UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

October 17, 1979

Docket No. 50-312

Mr. J. J. Mattimoe Assistant General Manager and Chief Engineer Sacramento Municipal Utility District 6201 S Street P. O. Box 15830 Sacramento, CA 95813

Dear Mr. Mattimoe:

RE: RANCHO SECO NUCLEAR GENERATING STATION

By letter dated July 18, 1979, you submitted a proposed inservice inspection and testing program description and a request for relief from selected ASME Code requirements pursuant to 10 CFR 50.55a(g). Although we have not completed our detailed review of your submittal, our preliminary review makes clear to us that your proposed program to implement those ASME Code requirements that you have found to be practical would increase the scope of inservice inspection and testing for your facility beyond that currently required by your Technical Specifications. We have concluded that this upgrading of your inservice inspection and testing program will further enhance safety.

Based on our preliminary review, we agree with your determination that it is impractical within the limitations of design, geometry and materials of construction of components, for you to meet certain of the specified ASME Code requirements and that imposition of those requirements would result in hardships or unusual difficulties without a compensating increase in the level of quality or safety. Therefore, pursuant to 10 CFR 50.55a(g)(6)(i). we hereby grant relief, on an interim basis, pending completion of our detailed review, from those inservice inspection and testing requirements of the ASME Code that you have requested as modified by NRC staff comments on your proposal made during the working session held with your staff on October 3 and 4, 1979. Moreover, since the scope of the inservice inspection and testing will be increased by your proposed program, and the granting of this relief is based only on the impracticality of selected ASME Code requirements, we have determined that the relief granted neither increases the probability or consequences of accidents previously considered nor decreases safety margins and that, therefore, it does not involve a significant hazards consideration. Therefore, you are authorized to, and should proceed to, implement your proposed program as modified by NRC staff comments made during the working session (except where your current Technical Specifications are more restrictive).

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Within 30 days of receipt of this letter, you should submit to the NRC documentation of the modifications you will be implementing with your proposed program based on the comments received from the staff during the working session on October 3 and 4, 1979.

During the period between the date that the requirements of 10 CFR 50.55a(g) become effective for your facility and the date we complete our detailed review of your submittal you must comply with both your existing Technical Specifications and your proposed inservice inspection and testing program. In the event conflicting requirements arise for some components, you must comply with the more restrictive requirements (e.g., shorter inspection intervals, increased number of parameters measured). In other words, the granting of this relief from ASME Code requirements should not be interpreted to give you relief from any of the requirements in your existing Technical Specifications.

When our detailed review of your July 18, 1979 submittal is complete, we will: (1) issue final approval of your program (which may contain modifications resulting from the staff's final review), (2) grant relief from any ASME Code requirements that are determined to be impractical for your facility for the duration of the inspection interval and (3) issue appropriate changes to your Technical Specifications.

A copy of the Federal Register Notice related to this action is enclosed.

Sincerely,

Robert W. Reid, Chief

Operating Reactors Branch #4
Division of Operating Reactors

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Enclosure: Federal Register Notice

cc: w/enclosure See next page

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w/enclosure(s):

General Counsel

201 S Street

0. Box 15830

1 Cramento, California 95813

Sacramento County Soard of Supervisors 327 7th Street, Room 424 Sacramento, California 95814

Susiness and Municipal Department Sacramento City-County Library 828 I Street Sacramento, California 95814

Director, Technical Assessment Division Office of Radiation Programs (AW-459) U. S. Environmental Protection Agency Crystal Mall #2 Arlington, Virginia 20460

U. S. Environmental Protection Agency Region IX Office ATTN: EIS COORDINATOR 215 Fremont Street San Francisco, California 94111

Mr. Fobert B. Borsum
Babcock & Wilcox
Muclear Power Generation Division
Suite 420, 7735 Old Georgetown Road
Eathesda, Maryland 20014

Herbert H. Brown, Esq.
Lowrence Coe Lanpher, Esq.
8511. Christopher and Phillips, P. C.
1900 M St., NW
Washington, D. C. 20036

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Christopher Ellison, Esq.
Dian Grueuich, Esq.
California Energy Commission
1111 Howe Avenue
Sacramento, California 95825

Ms. Eleanor Schwartz California State Office 600 Pennsylvania Avenue, S.E., Rm. 201 Washington, D.C. 20003

Docketing and Service Section Office of the Secretary U. S. Nuclear Regulatory Commission Washington, D.C. 20555

Michael L. Glaser, Esq. 1150 17th Street, N.W. Washington, D.C. 20036

Dr. Richard F. Cole
Atomic Safety and Licensing Board
Panel
U. S. Nuclear Regulatory Commission
Washington, D.C. 20555

Mr. Frederick J. Shon
Atomic Safety and Licensing Board
Panel
U. S. Nuclear Regulatory Commission
Washington, D.C. 20555

Timothy V. A. Dillon Esq. Suite 380 1850 K Street, N.W. Washington, D.C. 20006

James S. Reed, Esq.
Michael H. Remy, Esq.
Reed, Samuel & Remy
717 K Street, Suite 405
Sacramento, California 95814

Mr. Michael R. Eaton Energy Issues Coordinator Sierra Club Legislative Office 1107 9th St., Room 1020 Sacramento, CA 95814 Sacramento Municipal Utility District

cc w/enclosure(s):

Atomic Safety and Licensing Board Panel U. S. Nuclear Regulatory Commission Washington, D.C. 20555

Atomic Safety and Licensing Appeal Board Panel U. S. Nuclear Regulatory Commission Washington, D.C. 20555

Mr. Richard D. Castro 2231 K Street Sacramento, California 95814

Mr. Gary Hursh, Esq. 520 Capital Mall Suite 700 Sacramento, California 95814

California Department of Health ATTN: Chief, Environmental Radiation Control Unit Radiological Health Section 714 P Street, Room 498 Sacramento, California 95814