



UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION IV
611 RYAN PLAZA DRIVE, SUITE 1000
ARLINGTON, TEXAS 76011

7/2/79
DARA

April 21, 1979

In Reply Refer To:
RIV
Docket No. STN 50-482/IE Bulletin 79-05B

Kansas Gas and Electric Company
ATTN: Mr. Glenn L. Koester
Vice President-Operations
P. O. Box 208
Wichita, Kansas 67201

Gentlemen:

The enclosed IE Bulletin 79-05B is forwarded to you for information. No written response is required. We have also enclosed copies of recommendations of the ACRS to the Commission for your information. If you desire additional information regarding this matter, please contact this office.

Sincerely,

Karl V. Seyfrit
Karl V. Seyfrit
Director

Enclosures:

1. IE Bulletin No. 79-05B
with enclosures
2. ACRS Recommendations to the
Commission dated April 18,
1979 and April 20, 1979
3. List of IE Bulletins Issued
in the Last Twelve Months

cc w/enclosures:

Messrs. Nicholas A. Petrick, SNUPPS
D. T. McPhee, Kansas City Power and Light Company
Gerald Charnoff, Shaw, Pittman, Potts & Trowbridge
E. W. Creel, Kansas Gas and Electric Company

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
OFFICE OF INSPECTION AND ENFORCEMENT
WASHINGTON, DC 20555

APRIL 21, 1979

IE Bulletin 79-05B

NUCLEAR INCIDENT AT THREE MILE ISLAND - SUPPLEMENT

Description of Circumstances:

Continued NRC evaluation of the nuclear incident at Three Mile Island Unit 2 has identified measures in addition to those discussed in IE Bulletin 79-05 and 79-05A which should be acted upon by licensees with reactors designed by B&W. As discussed in Item 4.c. of Actions to be taken by Licensees in IEB 79-05A, the preferred mode of core cooling following a transient or accident is to provide forced flow using reactor coolant pumps.

It appears that natural circulation was not successfully achieved upon securing the reactor coolant pumps during the first two hours of the Three Mile Island (TMI) No. 2 incident of March 28, 1979. Initiation of natural circulation was inhibited by significant coolant voids, possibly aggravated by release of noncondensable gases, in the primary coolant system. To avoid this potential for interference with natural circulation, the operator should ensure that the primary system is subcooled, and remains subcooled, before any attempt is made to establish natural circulation.

Natural circulation in Babcock and Wilcox reactor systems is enhanced by maintaining a relatively high water level on the secondary side of the once through steam generators (OTSG). It is also promoted by injection of auxiliary feedwater at the upper nozzles in the OTSGs. The integrated Control System automatically sets the OTSG level setpoint to 50% on the operating range when all reactor coolant pumps (RCP) are secured. However, in unusual or abnormal situations, manual actions by the operator to increase steam generator level will enhance natural circulation capability in anticipation of a possible loss of operation of the reactor coolant pumps. As stated previously, forced flow of primary coolant through the core is preferred to natural circulation.

Other means of reducing the possibility of void formation in the reactor coolant system are:

- A. Minimize the operation of the the pressurizer and thereby reduce reduction by a blowdown through

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