



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 2 TO FACILITY OPERATING LICENSE NO. NPF-5

GEORGIA POWER COMPANY
OGLETHORPE ELECTRIC MEMBERSHIP CORPORATION
MUNICIPAL ELECTRIC ASSOCIATION OF GEORGIA
CITY OF DALTON, GEORGIA

EDWIN I. HATCH NUCLEAR PLANT UNIT NO. 2

DOCKET NO. 50-366

Introduction

By letter dated October 4, 1978, Georgia Power Company (the licensee) proposed a change to the Technical Specifications appended to Operating License No. NPF-5 for the Edwin I. Hatch Nuclear Plant Unit No. 2 (HNP-2). The change would provide for a one-time, temporary revision which would delete the Limiting Condition for Operation (LCO) on Primary Containment Internal Pressure to permit conduct of a test as part of the HNP-2 initial startup test program.

Evaluation

The purpose of the LCO on the maximum primary containment internal pressure of 0.75 psig is to ensure that the containment peak pressure will not exceed the design limit during Loss-of-Coolant-Accident conditions. This restriction, in conjunction with limits on leakage paths and rates assumed in the accident analysis, will limit the site boundary radiation doses to within the limits of 10 CFR Part 100 during accident conditions.

The licensee proposed that for purpose of conducting the Safety Relief Valve Discharge Capacity Startup Test, operation above the 0.75 psig limit temporarily is justified. His justification included the following bases:

1. Operation with a drywell pressure of 1.5 psig and a wetwell pressure of 0 psig is the same as operation of both drywell and wetwell at 0.75 psig since both conditions contain the same initial starting energy of the primary containment system.

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2. Operation above the 0.75 psig limit is expected to last less than 72 hours;
3. The test data would support the Commission's Mark I Long Term Program; and
4. Reactor power level would be less than 55% during the period which pressure is in excess of 0.75 psig.

We have reviewed the licensee's request and agree that the Temporary Technical Specification change does not significantly affect safety. The staff has independently determined that the peak containment pressure response as a function of initial conditions is relatively insensitive over the range of 0.7 to 2.0 psig (2.0 psig is the setpoint for ECCS actuation). We find this adequate support for relaxation of the LCO for primary containment pressure and the proposed temporary change is acceptable.

Environmental Considerations

We have determined that the amendment does not involve a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and pursuant to 10 CFR Section 51.5(d)(4) that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusions

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: October 13, 1978