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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

OCT 18 1979

Docket No. 50-358

Mr. Earl A. Borgmann
Vice President - Engineering
Cincinnati Gas & Electric Company
P. O. Box 960
Cincinnati, Ohio 45201

Dear Mr. Borgmann:

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION
(Wm. H. Zimmer, Unit No. 1)

In order that we may continue our review of your application for a license to operate the Zimmer Nuclear Power Station, Unit No. 1, your response to the enclosed request for additional information is needed. The request is based upon information contained in your application as amended through Revision 61 and your responses to our previous requests. We will need your response to this request prior to November 30, 1979.

Please contact us if you desire information or clarification regarding the enclosure.

Sincerely,

John F. Stolz

John F. Stolz, Chief
Light Water Reactors Branch No. 1
Division of Project Management

Enclosure:
Request for Additional
Information

cc: See next page

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Atomic Safety and Licensing Appeal
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ENCLOSURE

REQUEST FOR ADDITIONAL INFORMATION

WM. H. ZIMMER ROUND-TWO QUESTIONS

Introduction

This enclosure consists of the twenty-seventh in a series of positions (and requests for additional information). We will need your response in order to complete our safety evaluation of your Zimmer OL application. The request is in the area of:

212.0 Reactor Systems

It will be helpful to us if your responses are in a "Position and Response" format using the same number designation as the position. The first number designated the review area and the second (in parentheses) designated the associated section of the FSAR. Of course, your responses should include revision to the FSAR wherever appropriate.

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212.0 REACTOR SYSTEMS BRANCH

- 212.76 Provide transient curves of the following for the .017 ft² break in the HPCS line combined with a LPCS D/G failure and no LPCI diversion
- (a) water level inside shroud
 - (b) vessel pressure
 - (c) heat transfer coefficient
 - (d) peak cladding temperature
 - (e) LPCI flow
- 212.77 Provide the same information as above for a 0.17 ft² break in the recirculation line with an HPCS failure for the cases of LPCI diversion and no LPCI diversion
- 212.78 With respect to your LPCI flow diversion analyses and those resulting from your evaluation of the above questions (Q212.77 and Q212.78), provide an explanation of the various heat sources, such as reactor decay heat, wall heat, in determining the froth level for those breaks.

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