NRC FORM 366
(7-77)

U. S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT

CONTROL BLOCK: | | | | | | | (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1	V A N A S 1 (2)	0 0 - 0 0 0 0 0 - 0 0 (3)	4 1 1 1 1 (4)		(5)
	LICENSEE CODE	LICENSE NUMBER	LICENSE TYPE	CAT	

CON'T 0 1	REPORT SOURCE L (6)	0 5 0 0 0 3 3 8 (7)	0 9 2 5 7 9 (8)	1 0 2 3 7 9 (9)	
		DOCKET NUMBER	EVENT DATE	REPORT DATE	

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 While to Mode 3, operation personnel noted the steam driven auxiliary feedwater
0 3 pump 1-FW-P-2 was turning due to a steam leak thru a steam inlet trip valve.
0 4 The steam supply to 1-FW-P-2 was isolated using manually operated isolation
0 5 valves rendering it inoperable. This event is contrary to T.S. 3.7.1.2 and
0 6 reportable pursuant to T.S. 6.9.1.9.b. Two half-size auxiliary feedwater pumps
0 7 were operational; therefore the public health and safety were not affected by
0 8 this event.

	SYSTEM CODE	CAUSE CODE	CAUSE SUBCODE	COMPONENT CODE	COMP. SUBCODE	VALVE SUBCODE
0 9	H B (11)	E (12)	B (13)	V A L V E X (14)	F (15)	B (16)

(17) LER/RO REPORT NUMBER	EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE	REPORT TYPE		REVISION NO.
	7 9	—	1 3 0	/	0 3	L	—	0

ACTION TAKEN	FUTURE ACTION	EFFECT ON PLANT	SHUTDOWN METHOD	HOURS (22)	ATTACHMENT SUBMITTED	NPRD-4 FORM SUB.	PRIME COMP. SUPPLIER	COMPONENT MANUFACTURER
Z (18)	Z (19)	Z (20)	Z (21)	0 0 0 0	Y (23)	N (24)	A (25)	F 1 3 0 (26)

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 Isolation of steam to 1-FW-P-2 became necessary due to a leak thru a steam inlet
1 1 trip valve. The plant was in the Hot Shutdown condition (Mode 4) within 14 hours
1 2 of the event due to a shutdown that was in progress for a planned refueling outage.
1 3 Therefore, the dictates of the action statement of T.S. 3.7.1.2 were met. The
1 4 valve will be repaired during the refueling outage.

	FACILITY STATUS	% POWER	OTHER STATUS (30)	METHOD OF DISCOVERY	DISCOVERY DESCRIPTION (32)
1 5	X (28)	0 0 0 (29)	Emergency Shutdown	A (31)	Operator Observation

	ACTIVITY RELEASED	CONTENT OF RELEASE	AMOUNT OF ACTIVITY (35)	LOCATION OF RELEASE (36)
1 6	Z (33)	Z (34)	NA	NA

PERSONNEL EXPOSURES

	NUMBER	TYPE	DESCRIPTION (39)
1 7	0 0 0 (37)	Z (38)	NA

PERSONNEL INJURIES

	NUMBER	DESCRIPTION (41)
1 8	0 0 0 (40)	NA

1240 339

LOSS OF OR DAMAGE TO FACILITY (43)

	TYPE	DESCRIPTION
1 9	Z (42)	NA

PUBLICITY

7910300210

	ISSUED	DESCRIPTION (45)	NRC USE ONLY
2 0	N (44)	NA	

NAME OF PREPARER W. R. Cartwright PHONE: 703-894-5151

GPO 917-926

Virginia Electric and Power Company
North Anna Power Station, Unit #1
Docket No. 50-338
Report No. LER 79-130/03L-0

Attachment: Page 1 of 1

Description of Event

On September 25, 1979, steam was observed leaking past the seat of trip valve TV-MS111B. During Mode 3 operation, TV-MS-111B was isolated for repair. This valve is located on the steam line leading to the steam driven steam generator auxiliary feedwater pump 1-FW-P-2.

Probable Consequences of Occurrence

The steam supply line to the auxiliary feedwater pump required isolation thereby bringing into effect the ACTION statement of T.S. 3.7.1.2. This calls for one of three auxiliary feedwater pumps to be powered by an operable steam supply within 72 hours or be in Hot Shutdown within the next 12 hours. Since both motor driven auxiliary feed pumps were available for operation, the health and safety of the general public were not affected by this event. Potential generic implications for Unit 2 exist because of past problems encountered with these valves.

Cause of Occurrence

The steam leak was caused by the valve not seating properly.

Immediate Corrective Action

The unit was taken to Cold Shutdown for a scheduled refueling outage.

Scheduled Corrective Action

The valve will be repaired during the refueling outage.

Actions Taken To Prevent Recurrence

Causes of failure are being evaluated. The vendor has been contacted and his recommendation will be evaluated and implemented, as appropriate, during the refueling outage.

1240 340