

U.S. NUCLEAR REGULATORY COMMISSION  
CERTIFICATE OF COMPLIANCE  
For Radioactive Materials Packages

1.(a) Certificate Number	1.(b) Revision No.	1.(c) Package Identification No.	1.(d) Pages No.	1.(e) Total No. Pages
6386	5	USA/6386/AF	1	3

2. PREAMBLE

- 2.(a) This certificate is issued to satisfy Sections 173.393a, 173.394, 173.395, and 173.396 of the Department of Transportation Hazardous Materials Regulations (49 CFR 170-189 and 14 CFR 103) and Sections 146-19-10a and 146-19-100 of the Department of Transportation Dangerous Cargoes Regulations (46 CFR 146-149), as amended.
- 2.(b) The packaging and contents described in item 5 below, meets the safety standards set forth in Subpart C of Title 10, Code of Federal Regulations, Part 71, "Packaging of Radioactive Materials for Transport and Transportation of Radioactive Material Under Certain Conditions."
- 2.(c) This certificate does not relieve the consignor from compliance with any requirement of the regulations of the U.S. Department of Transportation or other applicable regulatory agencies, including the government of any country through or into which the package will be transported.

3. This certificate is issued on the basis of a safety analysis report of the package design or application—

3.(a) Prepared by (Name and address): U.S. Department of Energy Division of Naval Reactors Washington, DC 20545	3.(b) Title and identification of report or application: Safety Analysis Report for 235R001 Shipping Container dated August 11, 1970, as supplemented.  3.(c) Docket No. 71-6386
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4. CONDITIONS

This certificate is conditional upon the fulfilling of the requirements of Subpart D of 10 CFR 71, as applicable, and the conditions specified in item 5 below.

5. Description of Packaging and Authorized Contents, Model Number, Fissile Class, Other Conditions, and References:

(a) Packaging

(1) Model No.: 235R001

(2) Description

The 235R001 shipping container structure is horizontal, having an oblong cross section and is fabricated from 0.104 inch thick carbon sheet steel. The container is 313 inches long and has a maximum weight of 4,640 pounds, empty. The oblong cross section dimensions are approximately 35.5 inches high by 33.0 inches wide. The container was originally designed to ship unirradiated fuel modules of the A1G/A4W type. Subsequently, the container has been adapted to ship A1W-3 fuel modules using a strongback, partial A1W-R3 fuel modules using module support assemblies, standard size or partial S8G fuel modules by use of a special frame assembly and cradle clamps, S3G-3 refueling modules using cell support assemblies, rodged or unrodged D1G fuel modules, rodged ASNPP fuel cells and rodged or unrodged D2W fuel cells. The loaded container maximum weight is 12,421 pounds.

(3) Drawings

The packaging is constructed in accordance with Container Research Corporation Drawings Nos. 235R001, Rev. C, 235R004, Rev. C, and 235R005, Rev. 0, and Westinghouse Electric Corporation Drawings Nos. 973D425, Rev. 1, 903E693, Rev. 3, Sheet 1, 2 and 3 of 3, and 947J076, Rev. 0.

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(b) Contents

(1) Type and form of material

Unirradiated fuel assemblies of the following type,

- (i) A1G and A4W reactor cell without upper mechanism and with control rod, leadscrew and shipping fixture installed on rodded type modules.
- (ii) A4W Reactor Test Assembly (RTA).
- (iii) A1W-3 module without upper mechanism and with control rod, leadscrew, and control rod holddown device installed on rodded type modules.
- (iv) A1W-R3 Partial Lower Module.
- (v) Standard size S8G reactor cluster with regular or substitute support adapters and regular control rods or Boral poison rods with control rod holddown devices installed. If only one cell is shipped per container, a dummy load shall be installed for balance.
- (vi) Partial size S8G reactor cluster with regular or substitute support adapters and regular control rods or Boral poison rods with control rod holddown devices installed. If only one cell is shipped per container, a dummy load shall be installed for balance.
- (vii) S3G-3 refueling cells, with a maximum of one first pass 0-1 reactor cell assembly per container.
- (viii) D1G fuel module, rodded.
- (ix) D1G removable fuel assembly (RFA), unrodded.
- (x) A1G fuel cluster, fueled end only of full A1G reactor cell, rodded.
- (xi) ASNPP fuel cell with control rod, and control rod holddown device installed.
- (xii) D2W new side or central fuel cells with control rod and control rod holddown device, or D2W new corner fuel cells, unrodded.

(2) Maximum quantity of material per package

- (i) One fuel assembly as described in 5(b)(1)(i), 5(b)(1)(ii), 5(b)(1)(iii) or 5(b)(1)(xi).

(2) Maximum quantity of material per package (Cont'd.)

(ii) Two fuel assemblies as described in 5(b)(1)(iv), 5(b)(1)(v), 5(b)(1)(vi), 5(b)(1)(vii), 5(b)(1)(viii), 5(b)(1)(x) or 5(b)(1)(xii).

(iii) Four fuel assemblies as described in 5(b)(1)(ix).

(c) Fissile Class

III

Maximum number of packages per shipment:

(1) For the contents described in 5(b)(1)(iii) and limited in 5(b)(2)(i):

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(2) For the contents described in 5(b)(1)(i), 5(b)(1)(ii), 5(b)(1)(iv), 5(b)(1)(v), 5(b)(1)(vi), 5(b)(1)(vii), 5(b)(1)(viii), 5(b)(1)(ix), 5(b)(1)(x), 5(b)(1)(xi), and 5(b)(1)(xii) and limited in 5(b)(2)(i), 5(b)(2)(ii), and 5(b)(2)(iii):

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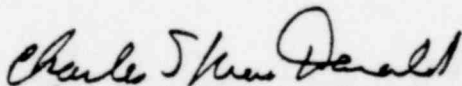
6. Expiration date: February 28, 1983.

REFERENCES

Safety Analysis Report for 235R001 Shipping Container, WAPD-OP(R)RD-357 dated August 11, 1970.

Supplements: Knolls Atomic Power Laboratory letter AIG 25-159; dated October 2, 1970, Bettis Atomic Power Laboratory letters WAPD-OP(R)RD-444; dated October 9, 1970, WAPD-OP(R)RD-476; dated October 26, 1970, and WAPD-OP(R)RD-488; dated 30, 1970, Knolls Atomic Power Laboratory letters AIG 25-181; dated April 9, 1971, and AIG 25-191; dated May 11, 1971, Bettis Atomic Power Laboratory letters WAPD-OP(R)C-94; dated May 16, 1972, WAPD-OP(R)C-199; dated December 13, 1972, and WAPD-OP(R)C-229; dated March 6, 1973, Naval Reactors letters G#5078; dated January 26, 1976, G#5776; dated September 8, 1977, G#5905; dated January 23, 1978, G#5923; dated February 22, 1978, G#6095; dated August 17, 1978, G#6208; dated March 8, 1979, and G#6373; dated September 4, 1979.

FOR THE U.S. NUCLEAR REGULATORY COMMISSION

  
Charles E. MacDonald, Chief  
Transportation Certification Branch  
Division of Fuel Cycle and  
Material Safety

DEC 03 1979

Dated: \_\_\_\_\_

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