NRC FOR (7-72)	U. S. NUCLEAR HEGULATORY COMMISSION	
•	LICENSEE EVENT REPORT	
	CONTROL BLOCK:	
	A L J M F 1 2 0 0 - 0 0 0 - 0 0 3 4 1	
	REPORT L 6 0 5 0 0 0 3 4 8 0 1 1 0 8 7 9 8 1 2 0 7 7 9 9 9 SOURCE 50 61 DOCKET NUMBER 68 69 EVENTE 74 75 REPORT DATE 80	
02	At 2210 on 11/8/79 makeup to the reactor coolage stem during a routine plant	
03	cooldown for maintenance (Mode 3 to 5), resulted in decreasing the available borated	
04	water volume in the Boric Acid Storage Tank (BAT) below the Tech. Spec. 3.1.2.8.a.1	
05	limit of 11,336 gallons by 442 gallons. An additional 6000 gallons of 8234 PPM boric	
06	acid solution was available but it could not be considered in meeting the Tech. Spec. J	
07	minimum due to an upper limit of 7700 PPM in the Tech. Spec. Tech. Spec. 3.1.2.8	
08	action statement requirements were met.	
09	$\begin{array}{c} \begin{array}{c} \text{SYSTEM} \\ \text{CODE} \\ \end{array} \\ \begin{array}{c} \text{CODE} \\ \text{ODE} \end{array} \\ \begin{array}{c} \text{CODE} \\ \text{SUBCODE} \end{array} \\ \begin{array}{c} \text{CAUSE} \\ \text{SUBCODE} \end{array} \\ \begin{array}{c} \text{COMPONENT CODE} \\ \text{COMPONENT CODE} \end{array} \\ \begin{array}{c} \text{COMPONENT CODE} \\ \text{SUBCODE} \end{array} \\ \begin{array}{c} \text{SUBCODE} \\ \text{SUBCODE} \end{array} \\ \end{array} $ \\ \begin{array}{c} \text{SUBCODE} \\ \text{SUBCODE} \end{array} \\ \begin{array}{c} \text{SUBCODE} \\ \text{SUBCODE} \end{array} \\ \end{array} \\ \begin{array}{c} \text{SUBCODE} \\ \text{SUBCODE} \end{array} \\ \end{array} \\ \begin{array}{c} \text{SUBCODE} \\ \text{SUBCODE} \end{array} \\ \end{array} \\ \begin{array}{c} \text{SUBCODE} \\ \text{SUBCODE} \end{array} \\ \end{array} \\ \begin{array}{c} \text{SUBCODE} \\ \text{SUBCODE} \end{array} \\ \end{array} \\ \end{array} \\ \begin{array}{c} \text{SUBCODE} \\ \end{array} \\ \end{array} \\ \begin{array}{c} \text{SUBCODE} \\ \text{SUBCODE} \end{array} \\ \end{array} \\ \end{array} \\ \begin{array}{c} \text{SUBCODE} \\ \ \end{array} \\ \end{array} \\ \end{array} \\ \begin{array}{c} \text{SUBCODE} \\ \end{array} \\ \end{array} \\ \end{array} \\ \begin{array}{c} \text{SUBCODE} \\ \ \end{array} \\ \end{array} \\ \end{array} \\ \begin{array}{c} \text{SUBCODE} \\ \end{array} \\ \end{array} \\ \begin{array}{c} \text{SUBCODE} \\ \end{array} \\ \end{array} \\ \end{array} \\ \end{array} \\ \begin{array}{c} \text{SUBCODE} \\ \end{array} \\ \end{array} \\ \end{array} \\ \begin{array}{c} \text{SUBCODE} \\ \end{array} \\ \end{array} \\ \end{array} \\ \end{array} \\ \\ \begin{array}{c} \text{SUBCODE} \\ \end{array} \\ \end{array} \\ \\ \end{array} \\ \begin{array}{c} \text{SUBCODE} \\ \end{array} \\ \end{array} \\ \\ \end{array} \\ \begin{array}{c} \text{SUBCODE} \\ \end{array} \\ \end{array} \\ \end{array} \\ \end{array} \\ \\ \\ \\	
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10	See Attachment.	
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	EASED OF RELEASE AMOUNT OF ACTIVITY 35 LOCATION OF RELEASE 36	
17	PERSONNEL EXPOSURES NUMBER TYPE DESCRIPTION 39 0 0 0 37 Z 38 NA	
1 3	PERSONNEL INJURIES NUMBER DESCRIPTION (4) 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0	
, 8 19	11 12 055 OF OH DAMAGE TO FACILITY (43) 7912120 YPE Description Z 42 NA 7912120	
20	PUBLICITY SUED DESCRIPTION (45) N (44) NA NA	926-
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LER 79-055/03L-0

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS

Makeup to the reactor coolant system on 11/8/79 due to the plant cooldown resulted in decreasing the B BAT level from 54% to 50%. 52.2% level is required in a single BAT to satisfy the Tech. Spec. limit of 11,336 gallons. BAT A was unavailable for consideration in meeting the Tech. Spec. because it contained borated water in excess of the Tech. Spec. limit of 7700 PPM due to batching that was in progress. The minimum temperature for solubility in the A BAT was not exceeded during the batching operation. The A BAT concentration was returned to less than 7700 PPM (7504) by 1050 on 11/9/79. The additional borated water source required by Tech. Spec. 3.1.2.8 (the RWST) was operable at all times during this event.

The basis for Tech. Spec. 3.1.2.8 states, in part, that the boration capability of the system is sufficient to provide a shutdown margin from expected operating conditions of $1.0\% \Delta k/k$ after xenon decay and cooldown to 200° F. It should be noted that when the plant began its cooldown, the Tech. Spec. 3.1.2.8 requirements were met and therefore, bases of Tech. Spec. 3.1.2.8 were met during the cooldown.

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