



72

UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION I
631 PARK AVENUE
KING OF PRUSSIA, PENNSYLVANIA 19406

Docket No. 50-333

MAY 13 1978

Power Authority of the State of New York
James A. FitzPatrick Nuclear Power Plant
ATTN: Mr. J. D. Leonard
Resident Manager
P. O. Box 41
Lycoming, New York 13093

Gentlemen:

The enclosed IE Information Notice No. 79-27 provides information with regard to the sequence of events that followed incidents involving steam generator tube ruptures at two PWR units. If you have any questions regarding this matter, please contact this office.

Sincerely,

Boyce H. Grier
Boyce H. Grier
Director

Enclosures:

1. IE Information Notice No. 79-27
2. List of Recently Issued Information Notices

CONTACT: W. H. Baunack
(215-337-5334)

cc w/encls:

George T. Berry, Executive Director
P. W. Lyon, Manager - Nuclear Operations
A. Klausmann, Director, Quality Assurance
M. C. Cosgrove, Quality Assurance Supervisor
J. F. Davis, Chairman, Safety Review Committee
V. J. Cassan, Assistant General Counsel
G. M. Wilverding, Licensing Supervisor

1503 051

7912050278

ENCLOSURE 1

SSINS No.: 6870
Accession No.:
7910250488

UNITED STATES
NUCLEAR REGULATORY COMMISSION
OFFICE OF INSPECTION AND ENFORCEMENT
WASHINGTON D.C. 20555

IE Information Notice No. 79-27
Date: November 16, 1979
Page 1 of 9

STEAM GENERATOR TUBE RUPTURES AT TWO PWR PLANTS

Description of Circumstances:

In recent months two incidents involving steam generator tube ruptures have occurred. In both instances, the units were cooled down and placed in the residual heat removal mode with existing procedures.

Event of June 25, 1979 at the Doel 2 Nuclear Power Plant in Belgium

The first event occurred on June 25, 1979, at the Doel 2 nuclear power plant in Belgium. The Doel unit is a 390 Mwe Westinghouse two-loop reactor. The event consisted of a rupture of several tubes in the loop B steam generator. The resultant leakage between the primary and secondary systems was estimated to be 125 gpm. The event started when the plant was heated up after a shutdown caused by a malfunction of the main steam isolation valve. At the time of the incident the primary coolant pressure was: 2233 psi and the temperature: 491°F. The reactor remained subcritical throughout the event.

The first indication of abnormal behavior was a rapid decrease of the primary system pressure (approximately: 28 psi/min.). This was followed by the sequence of events listed below:

	<u>Time, min.</u>
1. Increase of charging flow demand, requiring startup of a second charging pump.	1.8
2. Automatic isolation of the CVCS letdown line.	2.4
3. Shut off of the pressurizer heaters due to low liquid level in the pressurizer.	2.4

1503 052

DUPLICATE DOCUMENT

Entire document previously
entered into system under:

ANO 7910250488

No. of pages: 10