

ARKANSAS FOWER & LIGHT COMPANY POST OFFICE BOX 551 LITTLE ROCK, ARKANSAS 72203 (501) 371-4000

November 14, 1979

1-119-7

Mr. Norman M. Haller, Director Office of Management and Program Analysis U. S. Nuclear Regulatory Commission Washington, D. C. 20555

> Subject: Arkansas Nuclear One-Unit 1 Docket No. 50-313 License No. DPR-51

Monthly Operating Report (File: 0520.1)

Gentlemen:

Attached find the Monthly Operating Report for October, 1979, for Arkansas Nuclear One - Unit 1. Also attached is the 30 EFPD interval report of Core Power distribution RMS values as required per our submittal of April 26, 1979, (Ref. 1-049-16).

Very truly yours,

David C. Trimble

David C. Trimble Manager, Licensing

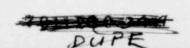
DCT/DVH/ew Attachments

oc: Mr. K. V. Seyfrit, Director
Office of Inspection and Enforcement
U.S. Nuclear Regulatory Commission
Region IV
611 Ryan Plaza Drive, Suite 1000
Arlington, Texas 76011

Mr. Victor Stello, Jr., Director Cffice of Inspection & Enforcement U. S. Nuclear Regulatory Commission Washington, D. C. 20555

1440 523

7911300329



DOCKET NO. __50-313 UNIT _ANO Unit-1 DATE 11/14/79 L. S. Bramlett COMPLETED BY TELEPHONE 501/968-2519

AVERAGE DAILY POWER LI (MWe-Net)	EVEL DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
823	17	827
821	18	826
825		765
824	20	. 2
824	21	0
824	22	0
826	23	0
825	24	0
825	25	0
826	26	0
825	27	0
823	28	0
824	29	0
828	30	0
828	31	0
827		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

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DUPE 7911200351



OPERATING DATA REPORT

DOCKET NO. 50-313 DATE 11/14/79 COMPLETED BY L.S. Bramlett TELEPHONE 501/968-2519

	OPERATING STATUS				
2. 3. 4. 5. 6. 7.	Unit Name: Arkansas Nuclear One Reporting Period: October 1-31, 19 Licensed Thermal Power (MWt): 2,568 Nameplate Rating (Gross MWe): 902, 7 Design Electrical Rating (Net MWe): 250 Maximum Dependable Capacity (Gross MWe): 4 Maximum Dependable Capacity (Net MWe): 4 If Changes Occur in Capacity Ratings (Items Nur None	Note- nce Last Report, Give Reasons:			
	Power Level To Which Restricted. If Any (Net M Reasons For Restrictions, If Any: NA				
		This Month	Yrto-Date	Cumulative	
		745.0	7296.0	42667.0	
	Hours In Reporting Period	457.2	3410.3	29120.4	
Dan.	Number Of Hours Reactor Was Critical	287.8	2257.5	4247.4	
	Reactor Reserve Shutdown Hours	456.8	3252.5	28466.2	
W. C.	Hours Generator On-Line	0.0	591.5	796.7	
	Unit Reserve Shutdown Hours	1160690.0	7970015.0	68776724.0	
	Gross Thermal Energy Generated (MWH) Gross Electrical Energy Generated (MWH)	391873.0	2646835.0	22887516.0	
	Net Electrical Energy Generated (MWH)	374828.0	2526664.0	21837210.0	
	Unit Service Factor	61.3	44.6	66.7	
	Unit Availability Factor	61.3	52.7	68.6	
	Unit Capacity Factor (Using MDC Net)	60.2	41.4	61.2	
	Unit Capacity Factor (Using DER Net)	59.2	40.7	60.2	
	Unit Forced Outage Rate	0.0	42.1	16.1	
	Shutdowns Scheduled Over Next 6 Months (Typ			1011	
	If Shut Down At End Of Report Period, Estimat Units In Test Status (Prior to Commercial Opera		November 15, 1979 Forecast	Achieved	
	INITIAL CRITICALITY				
	INITIAL ELECTRICITY				
	INITIAL ELECTRICITY				

1440 325

DATE: October 1979

REFUELING INFORMATION

Name of facility. Arkansas Nuclear One - Unit 1
Scheduled date for next refueling shutdown. 11/01/1980
Scheduled date for restart following refueling. 01/01/1981
Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? If answer is yes, what, in general, will these be? If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)?
Yes, Reload Report and associated proposed Technical Specification
Changes.
Scheduled date(s) for submitting proposed licensing action and
supporting information. 09/01/1980
new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures. Will reload 72 fresh fuel assemblies and operate for approximately 16 months.
The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool. a) 177 b) 176
The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies.
present 590 increase size by 0
The projected date of the last refueling that can be discharged
to the spent fuel pool assuming the present licensed capacity.
DATE:March, 1988

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH October

DOCKET NO. 50-313 UNIT NAME AND - Unit 1 DATE COMPLETED BY L. S. Bramlett TELEPHONE 501-968-2519

No.	Date	Type	Duration (Hours)	Reason?	Method of Shutting Down Reactor?	Licensee Eveni Report =	System Code4	Component	Cause & Corrective Action to Prevent Recurrence
79-05	791020	S	288.2	Н	1	NA	HH RC	PUMPXX INSTRU	Commitment to NRC to provide vital power to P7B, Emergency Feedwater Pump and modify In-Core Temperature Detection Devices.

F: Forced S: Scheduled Reason:

A-Equipment Failure (Explain) B-Maintenance of Test

C-Refueling

D-Regulatory Restriction

E Operator Training & License Examination

F-Administrative

G-Operational Error (Explain)

H Other (Explain)

3 Method:

1-Manual

2-Manual Scram.

3-Automatic Scram.

4-Other (Explain)

Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) Fac (NUREG-

01611

Exhibit 1 - Same Source

(9,77)

NRC MONTHLY OPERATING REPORT

OPERATING SUMMARY - OCTOBER, 1979

UNIT I

The unit operated at 100% reactor power for the first 19 days of the month. On October 20, the unit was taken off line for outage No. 79-05. This outage was scheduled for the provision of vital power to P7B Emergency Feedwater Pump and modification of In-Core Temperature Detection Devices. The unit remained in the shutdown mode for the remainder of the month.

ARKANSAS NUCLEAR ONE - UNIT I Periodic Core Power Distribution Comparison

A Radial Power Distribution comparison was performed at 66.4 EFPD. The RMS (root mean square) of the differences between measured and predicted at the 52 instrumented fuel assembly locations was 0.029 which is well within the acceptance criterion of 0.073.