METROPOLITAN EDISON COMPANY
JERSEY CENTRAL POWER & LIGHT COMPANY

AND

PENNSYLVANIA ELECTRIC COMPANY

THREE MILE ISLAND NUCLEAR STATION UNIT 1

Operating License No. DPR-50
Docket No. 50-289
Technical Specification Change Request No. 77

This Technical Specification Change Request is submitted in support of Licensee's request to change Appendix A to Operating License No. DPR-50 for Three Mile Island Nuclear Station Unit 1. As a part of this request, proposed replacement pages for Appendix A are also included.

METROPOLITAN EDISON COMPANY

By _____
Vice President

Sworn and subscribed to me this 30th day of August, 1978.

_____
Notary Public

GEORGE J. TROFFER
Notary Public, Reading, Berks Co.
My Commission Expires Jan. 20, 1980

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UNITED STATES OF AMERICA

NUCLEAR REGULATORY COMMISSION

IN THE MATTER OF

DOCKET NO. 50-289
LICENSE NO. DPR-50

METROPOLITAN EDISON COMPANY

This is to certify that a copy of Technical Specification Change Request No. 77 to Appendix A of the Operating License for Three Mile Island Nuclear Station Unit 1, has, on the date given below, been filed with the U. S. Nuclear Regulatory Commission and been served on the chief executives of Londonderry Township, Dauphin County, Pennsylvania and Dauphin County, Pennsylvania by deposit in the United States mail, addressed as follows:

Mr. Weldon B. Arehart
Board of Supervisors of
Londonderry Township
R. D. #1, Geyers Church Road
Middletown, Pennsylvania 17057

Mr. Harry B. Reese, Jr.
Board of County Commissioners
of Dauphin County
Dauphin County Court House
Harrisburg, Pennsylvania 17120

METROPOLITAN EDISON COMPANY

By _____
Vice President

Dated: August 30, 1978

1505 211

Three Mile Island Nuclear Station, Unit 1
Docket No. 50-289
Operating License No. DPR-50

Technical Specification Change Request No. 77

The licensee requests that the attached changed pages replace pages 4-36c through 4-36g, figures 4.4-1 through 4.4-5 and page 6-18.

Reasons for Change

This change request is based on the Commission's letter of December 29, 1977, which granted permission to modify the Ring Girder Surveillance Test.

Safety Evaluation Justifying Change

In their letter of December 29, 1977, the Commission requested to be informed of the repairs committed to in our letter of June 3, 1977. These repairs have been completed in strict accordance with the repair procedure for the cleaning, packing and curing of exposed concrete voids adjacent to dome bearing plates.

The Commission stated that, as a result of their review of the previous Reactor Containment Building Ring Girder Surveillance Tests, the safety margins in the structure at the present time, are adequate.

The Commission has concluded that there is reasonable assurance that the containment structure will continue to perform its intended safety function.

1505 212

4.4.2.2 Ringer Girder Inspection Program

The Ring Girder Inspection Program has been modified (per NRC letter of December 29, 1977) to consist of a visual inspection of the ring girder.

A. Periodic Inspection Schedule

The frequency of the modified ring girder inspections shall be in conjunction with the Inservice Tendon Surveillance Program per T.S. 4.4.2.2.1.

B. Inspection Method

The concrete adjacent to each tendon bearing plate on the ring girder shall be inspected visually. The acceptance criteria shall be that all crack widths greater than 0.010 inch shall be recorded and evaluated. Any crack width greater than 0.050 inch shall be cause for investigation to determine the amount of structural impairment upon the reactor building and its continued structural integrity.

Pages 4-36d through 4-36g and figures 4.4-1 through 4.4-5 deleted

1505 213

6.9 REPORTING REQUIREMENTS

6.9.3 Unique Reporting Requirements

A. Special reports shall be submitted to the Director of the Regulatory Operations Regional Office within the time period specified for each report. These reports shall be submitted covering the activities identified below:

Tests	Submittal Dates
(1) Containment Structural Integrity Test	
(a) Tendon Surveillance Program	Within 3 months after performance of surveillance program.
(2) Containment Integrated Leak Rate Test	Within 6 months after completion of test.
(3) Inservice Inspection Program	Within 6 months after five years of operation.

FOOTNOTES

1. A single submittal may be made for a multiple unit station. The submittal should combine those sections that are common to all units at the station.

2. This tabulation supplements the requirements of 20.407 of 10 CFR Part 20.

3. These reporting requirements apply only to Appendix A technical specifications.

1505 214