

LICENSEE EVENT REPORT

CONTROL BLOCK: _____ (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01 NYJAFI 200-0000-0000 341111 45
7 8 9 14 15 25 26 57 58 59

CON'T
01 REPORT SOURCE L 015000333 7092779 8101079 9
7 8 60 68 69 74 75 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCE (11)

02 Please See Attachment
03
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05
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07
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09 SYSTEM CODE SF 11 CAUSE CODE B 12 CAUSE SUBCODE C 13 COMPONENT CODE SUPPORT 14 COMP. SUBCODE A 15 VALVE SUBCODE Z 16
17 LER RO REPORT NUMBER 79 EVENT YEAR 79 SEQUENTIAL REPORT NO. 057 OCCURRENCE CODE 01 REPORT TYPE T REVISION NO. 0
ACTION TAKEN F 18 FUTURE ACTION Z 19 EFFECT ON PLANT Z 20 SHUTDOWN METHOD Z 21 HOURS 0000 ATTACHMENT SUBMITTED Y 23 NPD-4 FORM SUB. N 24 PRIME COMP. SUPPLIER A 25 COMPONENT MANUFACTURER S420 26

CAUSE DESCRIPTION AND CORRECTIVE ACTION (27)

10 Please See Attachment
11
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13
14

15 FACILITY STATUS E 28 % POWER 018 29 OTHER STATUS N/A 30 METHOD OF DISCOVERY D 31 A/E DISCOVERY DESCRIPTION 32

16 ACTIVITY CONTENT RELEASED OF RELEASE Z 33 Z 34 AMOUNT OF ACTIVITY N/A 35 LOCATION OF RELEASE 36

17 PERSONNEL EXPOSURES NUMBER 000 37 TYPE Z 38 DESCRIPTION N/A 39

18 PERSONNEL INJURIES NUMBER 000 40 DESCRIPTION N/A 41

19 LOSS OF OR DAMAGE TO FACILITY TYPE Z 42 DESCRIPTION N/A 43

20 PUBLICITY ISSUED DESCRIPTION N 44 N/A 45 7910150445 1151 263 NRC USE ONLY
NAME OF PREPARER W. Verne Childs PHONE: 315-342-3840

POWER AUTHORITY OF THE STATE OF NEW YORK
JAMES A. FITZPATRICK NUCLEAR POWER PLANT

DOCKET NO. 50-333

ATTACHMENT TO LER 79-057/01T-0

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During normal operation, as the result of the Pipe Stress Reanalysis being conducted by the Architect Engineer, the plant staff was notified that five pipe supports were considered inoperable; two were associated with Service Water return to the lake (H46-280 and H46-281) one was associated with Core Spray (H14-46) and two were associated with the RHR System (H10-503 and PFSK-503). As set forth in the August 14, 1979 letter lifting the Show Cause Order of March 13, 1979, analysis was performed to determine the effect of these overstressed supports upon system operability. From this analysis it was determined that two supports, H14-46 on the "B" Core Spray System and H46-281 on the Service Water System caused piping overstress.

The modifications to H14-46 were completed within the seven (7) day time frame required by the Technical Specifications and the August 14, 1979 letter. Surveillance testing of safety systems required by the Technical Specifications was satisfactorily accomplished during the period in which "B" Core Spray System was considered inoperable. The support is now fully acceptable.

The Service Water System is not addressed in the Technical Specifications. Modifications to H46-281 were accomplished within the seven (7) day time frame required by the August 14, 1979 letter. The support is now fully acceptable.

As set forth in Paragraph IV.4 of the August 14, 1979 letter, analysis was performed which proved the RHR System remained operable with H10-503 and PFSK-956 removed. Modifications to H10-503 have been completed. This support is now considered fully acceptable. Modification to PFSK-956 are in progress. The affected portion of the RHR system, the supply piping to the Fuel Pool Cooling System is isolated from the remainder of the system during power operations.

Analysis by the Architect Engineer showed that with H46-280 removed from the Service Water System no piping overstress would occur which would result in system inoperability. Modifications to this support are currently being designed. These will be pursued in an expeditious manner.

As noted above, all modification and analysis were performed within the time frames set forth in the August 14, 1979 letter and the Technical Specifications. All supports causing system inoperability have been upgraded such that the systems are fully operable. Modifications to those supports which, although inoperable, do not result in system inoperability, are being pursued in an expeditious manner. Therefore, the event did result in a significant safety hazard to the public health and safety.

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