POWER AUTHORITY OF THE STATE OF NEW YORK
JAMES A. FITZPATRICK NUCLEAR POWER PLANT

JOHN D. LEONARD, JR.
Resident Manager

P.O. BOX 41
Lycoming, New York 13093
315-342-3840

October 19, 1979
JAFP-79- 569

Mr. Boyce H. Grier
United States Nuclear Regulatory Commission
Region 1
631 Park Avenue
King of Prussia, Pennsylvania 19406

Reference: Docket No. 50-333 Licensee Event Report: 79/066/03X-1

Dear Mr. Grier:

We have enclosed the referenced Licensee Event Report in accordance with Section 6.0 of Technical Specifications and USNRC Regulatory Guide 1.16.

If there are any questions concerning this report, please contact Mr. W. Verne Childs at 315-342-3840, Extension 207.

Very truly yours,

John D. Leonard, Jr.
Resident Manager

JDL:VC:jjh
Enclosure

CC: USNRC Director, Office of Inspection & Enforcement (30 copies)
USNRC Director, Office of Management Information & Program Control (3 copies)
Internal Power Authority Distribution

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ATTACHMENT TO LER 79-066/03X-1 Page 1 of 1

During normal operation while conducting Operations Surveillance Test F-ST-5Q titled "Flow Bias Functional Test" to satisfy the requirements of Technical Specifications Appendix A, Table 3.1-1, Average Power Range Monitor (APRM) Channels A, D, and F tripped at a value of 120.5, as read by the Operator on the meter face. This was .5% over the Technical Specification requirement of equal to or less than 120% of rated power.

Operations personnel bypassed Channel "A" and inserted a half scram into the "B" side of the RPS System to meet the requirements of Technical Specifications. Plant operation continued in this manner until Instrumentation and Control Technicians investigated the problem on the following day. The investigation did not reproduce the out of tolerance values noted earlier which apparently were due to the Operator's misreading the meter face by .5%. Retest of the instruments, utilizing F-ST-5Q, the identical Surveillance Test the Operator had used, revealed that the trip points were within the limits of Technical Specifications. In view of these findings, the half scram was removed from the "B" side of the reactor protection system and the APRM system was restored to normal. This was not an actual drift of a level set point. Therefore, it did not represent any significant hazard to the public health and safety.

NOTE: Revision 1 of this LER is submitted to clarify that no drift of the instrument set point took place.