# JERSEY CENTRAL POWER & LIGHT COMPANY OYSTER CREEK NUCLEAR GENERATING STATION

Provisional Operating License No. DPR-16

Technical Specification Change Request No. 74 Docket No. 50-219

Applicant submits, by this Technical Specification Change Request No. 74 to the Oyster Creek Nuclear Generating Station Technical Specifications, revising section 3.4.A.10 and section 3.5.A.2 concerning draining of the torus.

JERSEY CENTRAL POWER & LIGHT COMPANY

BY:

ice President

STATE OF NEW JERSEY COUNTY OF MORRIS

Sworn and subscribed to before me on this 24th day of Sept. , 1979.

PHYLLIS A. KABIS NOTARY PUBLIC OF NEW JERSEY

My Commission Expires Aug. 16, 1984

Hustin B. Kain

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# UNITED STATES OF AMERICA NUCLEAR REGULATORY COMMISSION

| IN THE        | MATTER  | OF )  |   |       |         |   |  |  |  |    |        |     |        |
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|               |         |       |   |       |         | ) |  |  |  |    | DOCKET | NO. | 50-219 |
| <b>JERSEY</b> | CENTRAL | POWER | 8 | LIGHT | COMPANY | ) |  |  |  | 41 |        |     |        |

## CERTIFICATE OF SERVICE

This is to certify that a copy of Technical Specification Change
Request No. 74 for the Oyster Creek Nuclear Generating Station Technical
Specifications, filed with the U. S. Nuclear Regulatory Commission on
September 24, has this 24th day of September, 1979, been served on the
Mayor of Lacey Township, Ocean County, New Jersey by deposit in the United
States mail addressed as follows:

The Honorable Marylou Smith Mayor of Lacey Township P. O. Box 475 Forked River, New Jersey 08731

JERSEY CENTRAL POWER & LIGHT COMPANY

BY:

DATED: September 24, 1979



Jersey Central Power & Light Company Madison Avenue at Punch Bowl Road Morristown, New Jersey 07960 (201) 455-8200

September 24, 1979

The Honorable Marylou Smith Mayor of Lacey Township P. O. Box 475 Forked iver, New Jersey 08731

Dear Mayor Smith:

Enclosed herewith is one copy of Technical Specification Change Request No. 74 for the Oyster Creek Nuclear Generating Station Technical Specifications.

These documents were filed with the U. S. Nuclear Regulatory Commission on September 24, 1979.

Very truly yours,

Ivan R. Finfro

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Enclosure

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JERSEY CENTRAL POWER & LIGHT COMPANY

OYSTER CREEK NUCLEAR GENERATING STATION

(DOCKET NO. 50-219)

PROVISIONAL OPERATING LICENSE NO. DPR-16

Applicant hereby requests the Commission to change Appendix A to the above captioned license as follows:

#### 1. Sections to be changed:

Section 3.4.A.10 and section 3.5.A.2.

# 2. Extent of changes:

Revise the above sections to allow draining the torus without having to meet the requirement of removing the reactor vessel head, fuel pool gate and separator-dryer gate and raising the water level above elevation 117 feet.

## 3. Changes requested:

Replace revised pages 3.4-1a, 3.4-1b, and 3.5-1a.

# 4. Discussion:

This Technical Specification change will revise Section 3.4 and 3.5 to allow draining of the suppression pool without having to remove the vessel head and fuel, separator-dryer pool gates.

To ensure an adequate water supply to the vessel, while the suppression pool is drained, certain prerequsites to draining the torus have to be met. These prerequisites include:

- a. An operable flow path capable of taking suction from the condensate storage tank and transferring water to the reactor vessel.
  - b. The fire protection system is operable.
- c. At least one core spray pump, and system components necessary to deliver rated core spray flow to the reactor vessel must remain operable to the extent that the pump and any necessary valves can be started or operated from the control room or from local control stations, and the torus is mechanically intact.
- d. The condensate storage tank level is maintained greater than thirty (30) feet (360,000 gallons).

In the unlikely event that a drain path is established which

would have the potential to drain the reactor vessel, the condensate storage tank would provide the necessary makeup volume needed to keep the core covered. Several flow paths may be used to transfer the water such as filling the hotwells from the condensate storage tank and using the condensate system to transfer this water to the reactor vessel or using the core spray system to transfer the water to the reactor vessel from the condensate storage tank. By requiring that the condensate storage tank level be maintained greater than thirty (30) feet will ensure that there is sufficient water inventory available, should the leak go unchecked, to fill the lower portion of the drywell, empty into the torus and fill the suction header of the core spray system (330,000 gallons). The core spray system can then be lined up to recirculate the water to the reactor vessel.

The fire protection system will be used as a backup source of makeup water should it be required. There is sufficient volume in the fire pond to provide the makeup water necessary to place the core spray system in operation to recirculate water to the reactor vessel.

To further minimize the possibility of draining the vessel, a requirement that no work shall be performed on the vessel which could result in lowering the reactor water level to less than 4'8" above the top of the active fuel will be in effect while the torus is drained unless the reactor vessel head, fuel pool gate, and separator-dryer pool gates are removed and the water level is above elevation 117 feet. This requirement will assure that there is a ready source of makeup water available while work is being performed which could have the potential of draining the vessel.

With alternate sources of water available and a requirement that no work be performed which could have the potential of draining the reactor vessel, this proposed change request will not increase the probability of an accident or risk to the health and safety of the public.

- a. At least one core spray pump, and system components necessary to deliver rated core spray to the reactor vessel, must remain operable to the extent that the pump and any necessary valves can be started or operated from the control room or from local control stations.
- b. The fire protection system is operable, and
- c. These systems are demonstrated to be operable on a weekly basis.
- 8. If necessary to accomplish maintenance or modifications to the core spray systems, their power supplies or water supplies, reduced system availability is permitted when the reactor is in the refuel mode with the reactor coolant system maintained at less than 212 F or in the startup mode for the purposes of low power physics testing. Reduced core spray system availability is defined as follows:
  - a. At least one core spray pump in each loop, and system components necessary to deliver rated core spray to the reactor vessel, must remain operable to the extent that the pump and any necessary valves in each loop can be started or operated from the control room or from local control stations.
  - b. The fire protection system is operable and,
  - c. Each core spray pump and all components in 3.4.A.8a are demonstrated to be operable every 72 hours.
- 9. If Specifications 3.4.A.7 and 3.4.A.8 cannot be met, the requirements of Specification 3.4.A.6 will be met and work will be initiated to meet minimum operability requirements of 3.4.A.7 and 3.4.A.8.
- 10. The core spray system is not required to be operable when the following conditions are met:
  - a. The reactor mode switch is locked in the "refuel" or "shutdown" position.
  - b.(1) There is an operable flow path capable of taking suction from the condensate storage tank and transferring water to the reactor vessel, and
    - (2) The fire protection system is operable.

- c. The reactor coolant system is maintained at less than 212 F and vented.
- d. At least one core spray pump, and system components necessary to deliver rated core spray flow to the reactor vessel, must remain operable to the extent that the pump and any necessary valves can be started or operated from the control room or from local control stations, and the torus is mechanically intact.
- e.(1) No work shall be performed on the reactor or its connected systems which could result in lowering the reactor water level to less than 4'8" above the top of the active fuel and the condensate storage tank level is greater than thirty (30) feet (360,000 gallons).

OR

(2) The reactor vessel head, fuel pool gate, and separator-dryer pool gates are removed and the water level is above elevation 117 feet.

NOTE: When filling the reactor cavity from the condensate storage tank and draining the reactor cavity to the condesate storage tank, the 30 foot limit does not apply provided there is sufficient amount of water to complete the flooding operation.

- B. Automatic Depressurization System
  - 1. Five electromatic relief valves of the automatic depressurization system shall be operable when the reactor is pressurized above 110 psig, except as specified in 3.4.B.2.

- c. The reactor coolant system is maintained at less than 212 F and vented.
- d. At least one core spray pump, and system components necessary to deliver rated core spray flow to the reactor vessel, must remain operable to the extent that the pump and any necessary valves can be started or operated from the control room or from local control stations, and the torus is mechanically intact.
- e.(1) No work shall be performed on the reactor or its connected systems which could result in lowering the reactor water level to less than 4'8" above the top of the active fuel and the condensate storage tank level is greater than thirty (30) feet (360,000 gallons).

OR

(2) The reactor vessel head, fuel pool gate, and separator-dryer pool gates are removed and the water level is above elevation 117 feet.

NOTE: When filling the reactor cavity from the condensate storage tank and draining the reactor cavity to the condensate storage tank, the 30 foot limit does not apply provided there is sufficient amount of water to complete the flooding operation.

3. Primary containment integrity shall be maintained at all times when the reactor is critical or when the reactor water temperature is above 212 F and fuel is in the reactor vessel except while performing low power physics tests at atmospheric pressure during or after refueling at power levels not to exceed 5 MWt.