### INTERIM REPORT

### 9-17-79

Accession No.

Contract Program or Project Title:

LOFT Experimental Program Division

RELAP4/MOD6 Prediction Comparisons with LOFT LOCE L2-3 Data

Type of Document: .

Subject of this Document:

LOFT Technical Report

Author(s):

C. D. Keeler, J. R. White

Date of Document:

August 1979

Responsible NRC Individual and NRC Office or Division:

This document was prepared primal sy for preliminary or internal use. It has not

received full review and approval. Since there may be substantive changes, this document should not be considered final.

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Prepared for U.S. Nuclear Regulatory Commission Washington, D.C. 20555

NRC Fin #A6048

INTERIM REPORT

NRC Research and Technical 995 294 Assistance Report

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## **RELAP4/MOD6 PREDICTION COMPARISONS** WITH LOFT LOCE L2-3 DATA

C.D. KEELER J.R. WHITE

August 1979





## LOFT TECHNICAL REPORT LOFT PROGRAM

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RELAP4/MOD6 PREDICTION COMPARISONS WITH LOFT LOCE	REPORT NO	
		LTR 20-104
C. D. Keeler and J. R. White	GNA NO.	
LOFT Experimental Program Division	DATE	August 16, 1979

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## RESOLUTION OF RECOMMENDATIONS

This LTR documents comparisons between experimental data and computer code analysis. There are no recommendations specific to LOFT plant configuration. Future analysis will be done to gain better understanding of modeling heat transfer processes.

> NRC Research and Technical Assistance Report



(Rev 12-78)

## **INTERIM REPORT**

Accession No	
Report No	LTR 20-104
Proj. No.	P 394

995 298

### **Contract Program or Project Title:**

LOFT Experimental Program Division

#### Subject of this Document:

RELAP4/MOD6 Prediction Comparisons with LOFT LOCE L2-3 Data

### Type of Document:

LOFT Technical Report

### Author(s):

C. D. Keeler and J. R. White

#### Date of Document:

August 1979

### Responsible NRC Individual and NRC Office or Division:

G. D. McPherson, Acting Chief, LOFT Research Branch, Division of Reactor Safety Research, USNRC

This document was prepared primarily for preliminary or internal use. It has not received full review and approval. Since there may be substantive changes, this document should not be considered final.

EG&G Idaho, Inc. Idal o Falls, Idaho 83401

Prepared for the U.S. Nuclear Regulatory Commission and the U.S. Department of Energy Idaho Operations Office Under contract No. DE-AC07-76ID01570 NRC FIN No. A6048

## INTERIM REPORT

NRC Research and Technical

Assistance Report

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#### SUMMARY

This document contains comparison between RELAP4/MOD6 predicted and experimental measured quantities for Loss-of-Coolant Experiment (LOCE) L2-3 performed in the Loss-of-Fluid Test (LOFT) facility. These data comparisons provide a detailed record for subsequent analysis.

Comparisons indicate that the trends in the system hydraulic response were generally well predicted. The core thermal response, in general, was not well predicted due to the code failing to predict the early rewet. It is recommended that future LOCE L2-3 posttest analysis efforts be centered on gaining a better understanding of modeling rewet phenomena with the RELAP4/MOD6 heat transfer surface. Better modeling of the performance of the steam generator secondary side is also needed before modeling small break LOCEs.

## CONTENTS

.

.

SUMM	ARY				•	•	•	•			ii
1.	INTRODUCTION					ĺ,					1
2.	DATA COMPARISONS	•			į.	•					2
	<ol> <li>Experimental Data Classifications .</li> <li>Experimental Parameters Compared</li> </ol>	:	::	::	:	:	:	:	:	:	2 4
3.	CONCLUSIONS AND RECOMMENDATIONS	•									60
4.	REF ERENCES										60
APPE SY ST	DIX A - EXPERIMENTAL MEASUREMENT LOCATION	S I	N TH	E L	.0F	т					61

## FIGURES

1.	Comparison of predicted and measured average dens broken loop cold leg (DE-BL-1)	ity	/ i	n •				7
2.	Comparison of predicted and measured average dens broken loop hot leg (DE-BL-2)	ity	/ i	n				7
3.	Comparison of predicted and measured average dens intact loop cold leg (DE-PC-1)	ity 	/ i	n				8
4.	Comparison of predicted and measured average dens intact loop hot leg (DE-PC-2)	ity 	( i)	n				8
5.	Comparison of predicted and measured differential across 14- to 5-in. reducer (PdE-BL-1)	pr	es:	sur	e •			9
6.	Comparison of predicted and measured differential across pump simulator (PdE-BL-5)	pr	es:	sur	e •			9
7.	Comparison of predicted and measured differential across steam generator simulator (PdE-BL-7)	pr	es:	sur	e •			10
8.	Comparison of predicted and measured differential across pumps (PdE-PC-1)	pr	es:	sur	e •			10
9.	Comparison of predicted and measured differential across intact loop steam generator (PdE-PC-2) .	pr	es:	sur	e •			11
10.	Comparison of predicted and measured differential across Pump 1 (PdE-PC-9)	pr	ess.	sur	e •			11
	iii	9	95	ó		3	20	2

11.	Comparison of predicted and measured differential pressure across Pump 2 (PdE-PC-10)	
12.	Comparison of predicted and measured flow rate from accumulator (FT-P120-36-1)	
13.	Comparison of predicted and measured flow rate from low- pressure injection system pump (FT-P120-85) 13	
14.	Comparison of predicted and measured flow rate from high- pressure injection system pump (FT-P128-104)	
15.	Comparison of predicted and measured liquid level in accumulator (LIT-P120-44)	
16.	Comparison of predicted and measured liquid level in steam generator secondary side (LT-P004-8B)	
17.	Comparison of predicted and measured liquid level in pressurizer Channel B (LT-P139-7)	
18.	Comparison of predicted and measured mass flow in broken loop cold leg (FR-BL-1)	
19.	Comparison of predicted and measured mass flow in broken loop hot leg (FR-BL-2)	
20.	Comparison of predicted and measured mass flow in intact loop cold leg (FR-PC-1)	
21.	Comparison of predicted and measured mass flow in intact loop hot leg (FR-PC-2)	
22.	Comparison of predicted and measured total ECC flow rate (FR-ECC-1)	
23.	Comparison of predicted and measured integral of ECC flow (MS-RV-001)	
24.	Comparison of predicted and measured momentum flux in broken loop cold leg (ME-BL-1)	
25.	Comparison of predicted and measured momentum flux in broken loop hot leg (ME-BL-2)	
26.	Comparison of predicted and measured momentum flux in intact loop cold leg (ME-PC-1)	i
27.	Comparison of predicted and measured momentum flux in intact loop hot leg (ME-PC-2)	
28.	Comparison of predicted and measured momentum flux at instrument Stalk 1 (ME-1ST-1) 20	

.

.

iv

29.	Comparison of predicted and measured momentum flux in upper end box (ME-1UP-1)	21
30.	Comparison of predicted and measured momentum flux in upper end box (ME-3UP-1)	21
31.	Comparison of predicted and measured momentum flux at instrument Stalk 2 (ME-2ST-1)	22
32.	Comparison of predicted and measured pressure in broken loop cold leg (PE-BL-1)	22
33.	Comparison of predicted and measured pressure in broken loop hot leg (PE-BL-2)	23
34.	Comparison of predicted and measured pressure in broken loop hot leg pump simulator outlet (PE-BL-3)	23
35.	Comparison of measured and predicted pressure at steam generator simulator outlet (PE-BL-6)	24
36.	Comparison of predicted and measured pressure in broken loop cold leg spool piece midpoint (PE-BL-8)	24
37.	Comparison of predicted and measured pressure in intact loop cold leg (PE-PC-1)	25
38.	Comparison of predicted and measured pressure in intact loop hot leg (PE-PC-2)	25
39.	Comparison of predicted and measured pressure in intact loop pressurizer (PE-PC-4)	26
40.	Comparison of measured and predicted pressure at instrument Stalk 1 (PE-1ST-1A)	26
41.	Comparison of predicted and measured pressure at instrument Stalk 1 (PE-1ST-3A)	27
42.	Comparison of predicted and measured pressure in upper end box (PE-1UP-1A)	27
43.	Comparison of predicted and measured pressure at instrument Stalk 2 (PE-2ST-1A)	28
44.	Comparison of predicted and measured pressure in blowdown suppression tank (PE-SV-17)	28
45.	Comparison of predicted and measured pressure in steam generator secondary side (PT-P004-10A)	29
46.	Comparison of predicted and measured pressure in accumulator (PT-P120-43)	29

۷

47.	Comparison of predicted and measured pressure at ECC cold leg injection point (PT-P120-61)	30
48.	Comparison of predicted and measured pump speed for Pump 1 (RPE-PC-1)	30
49.	Comparison of predicted and measured average velocity in broken loop cold leg (FE-BL-1)	31
50.	Comparison of predicted and measured average velocity in broken loop hot leg (FE-BL-2)	31
51.	Comparison of predicted and measured average velocity in intert loop cold leg (FE-PC-1)	32
52.	Comparison of predicted and measured average velocity in intact loop hot leg (FE-PC-2)	32
53.	Comparison of predicted and measured average coolant temperature in broken loop cold leg (TE-BL-1)	33
54.	Comparison of predicted and measured average coolant temperature in broken loop hot leg (TE-BL-2)	33
55.	Comparison of predicted and measured average coolant temperature in intact loop cold leg (TE-PC-1)	34
56.	Comparison of predicted and measured average coolant temperature in intact loop hot leg (TE-PC-2)	34
57.	Comparison of predicted and measured coolant temperature on instrument Stalk 1 (TE-1ST-1)	35
58.	Comparison of predicted and measured coolant temperature on instrument Stalk 1 (TE-1ST-2)	35
59.	Comparison of predicted and measured coolant temperature on instrument Stalk 1 (TE-1ST-3)	36
60.	Comparison of predicted and measured coolant temperature on instrument Stalk 1 (TE-1ST-4)	36
61.	Comparison of predicted and measured coolant temperature on instrument Stalk 1 (TE-1ST-5)	37
62.	Comparison of predicted and measured coolant temperature on instrument Stalk 1 (TE-1ST-6)	37
63.	Comparison of predicted and measured coolant temperature on instrument Stalk 1 (TE-1ST-8)	38
64.	Comparison of predicted and measured coolant temperature on instrument Stalk 1 (TE-1ST-9)	20
	vi 995 3	,05

65.	Comparison of on instrument	predicted and Stalk 1 (TE-1	d measured ST-11) .	coolant	temperature			39
66.	Comparison of on instrument	predicted and Stalk 1 (TE-1	d measured LST-12) .	coolant	temperature			39
67.	Comparison of on instrument	predicted and Stalk 1 (TE-1	1 measured LST-13) .	coolant	temperature			40
68.	Comparison of on instrument	predicted and Stalk 1 (TE-1	d measured ST-14) .	coolant	temperature			40
69.	Comparison of on instrument	predicted and Stalk 2 (TE-2	measured	coolant	temperature			41
70.	Comparison of on instrument	predicted and Stalk 2 (TE-2	measured	coolant	temperature			41
71.	Comparison of on instrument	predicted and Stalk 2 (TE-2	measured	coolant	temperature			42
72.	Comparison of on instrument	predicted and Stalk 2 (TE-2	measured	coolant	temperature		•	42
73.	Comparison of on instrument	predicted and Stalk 2 (TE-2	measured ST-7)	coolant	temperature			43
74.	Comparison of on instrument	predicted and Stalk 2 (TE-2	measured	coolant	temperature			43
75.	Comparison of on instrument	predicted and Stalk 2 (TE-2	measured	coolant	temperature			44
76.	Comparison of on instrument	predicted and Stalk 2 (TE-2	measured ST-13) .	coolant	temperature			44
77.	Comparison of on instrument	predicted and Stalk 2 (TE-2	measured ST-14) .	coolant	temperature			45
78.	Comparison of in lower end b	predicted and box (TE-1LP-1)	measured	coolant	temperature	2		45
79.	Comparison of in lower end b	predicted and box (TE-2LP-1)	measured	coclant	temperature			46
80.	Comparison of in lower end t	predicted and box (TE-3LP-1)	measured	coolant	temperature			46
81.	Comparison of in upper end b	predicted and box (TE-1UP-1)	measured	coolant	temperature			47
82.	Comparison of on drag disc-t	predicted and turbine transd	measured lucer FE-11	coolant JP-1 (TE-	temperature 1UP-5)			47

83.	Comparison of predicted and measured coolant temperature in upper end box (TE-2UP-1)
84.	Comparison of predicted and measured coolant temperature in upper end box (TE-3UP-1) 48
85.	Comparison of predicted and measured coolant temperature on drag disc-turbine transducer FE-3UP-1 (TE-3UP-5) 49
86.	Comparison of predicted and measured coolant temperature in upper end box (TE-4UP-1) 49
87.	Comparison of predicted and measured coolant temperature in upper end box (TE-5UP-1)
88.	Comparison of predicted and measured coolant temperature on drag disc-turbine transducer FE-5UP-1 (TE-5UP-9) 50
89.	Comparison of predicted and measured coolant temperature in upper end box (TE-6UP-1)
90.	Comparison of predicted and measured cladding temperature in fuel Assembly 5 (TE-5F4-15)
91.	Comparison of predicted and measured cladding temperature in fuel Assembly 5 (TE-5J4-15)
92.	Comparison of predicted and measured cladding temperature in fuel Assembly 5 (TE-5F4-21)
93.	Comparison of predicted and measured cladding temperature in fuel Assembly 5 (TE-5J4-21)
94.	Comparison of predicted and measured cladding temperature in fuel Assembly 5 (TE-5F4-26)
95.	Comparison of predicted and measured cladding temperature in fuel Assembly 5 (TE-5J4-26)
96.	Comparison of predicted and measured cladding temperature in fuel Assembly 5 (TE-5D6-30)
97.	Comparison of predicted and measured cladding temperature in fuel Assembly 5 (TE-5F4-30)
98.	Comparison of predicted and measured cladding temperature in fuel Assembly 5 (TE-5J4-30)
99.	Comparison of predicted and measured cladding temperature in fuel Assembly 5 (TE-5L6-30)
100.	Comparison of predicted and measured cladding temperature in fuel Assembly 5 (TE-5D6-32)

101.	Comparison of predicted and measured of in fuel Assembly 5 (TE-5L6-32)	cladding temperature
102.	Comparison of predicted and measured of in fuel Assembly 5 (TE-5D6-37)	cladding temperature
103.	Comparison of predicted and measured of in fuel Assembly 5 (TE-5L6-37)	cladding temperature
104.	Comparison of predicted and measured of in fuel Assembly 5 (TE-5D6-39)	cladding temperature
105.	Comparison of predicted and measured of in fuel Assembly 5 (TE-5L6-39)	cladding temperature
A-1.	Intact loop measurement locations	65
A-2.	Broken loop measurement locations	67
A-3.	Reactor vessel measurement locations .	69
A-4.	Reactor vessel upper plenum measuremen	nt locations 71
A-5.	Core map showing fuel rod position des	signations 73

1

## RELAP4/MOD6 PREDICTION COMPARISONS WITH LOFT LOCE L2-3 DATA

### 1. INTRODUCTION

An extensive comparison has been made between the experimental data from Loss-of-Coolant Experiment (LOCE)  $L2-3^1$ , performed in the Loss-of-Fluid Test (LOFT) facility, and data from the experimental prediction for LOCE  $L2-3^2$ , performed using a modified version of RELAP4/MOD6. The objective of this report is to make these comparisons generally available, with only limited qualitative observations, and to define areas of future posttest analysis.

The LOFT facility<sup>3</sup> is a 50 MW(t) volumetrically scaled pressurized water reactor (PWR) system designed to study the response of the engineered safety features in commercial PWR systems during the postulated loss-of-coolant accident. The LOFT system contains major components representative of those in a typical four-loop PWR and can be operated at conditions typical of a PWR. The LOFT reactor vessel contains a downcomer, a lower plenum, a nuclear core, and an upper head. The LOFT piping system consists of an intact loop, which represents the three unbroken loops of a four-loop PWR, and a broken loop, which represents a broken loop of a PWR.

LOCE L2-3 was a nuclear experiment simulating a 200% (100% break area in each leg of the LOFT broken loop) double-ended offset shear in the cold leg of a four-loop PWR. At experiment initiation the LOFT system was operating at a power level of 36.7 MW, which yielded a maximum linear heat generation rate of 39.4 kW/m, the mass flow rate in the intact loop was 199.8 kg/s, and the temperature of the primary coolant ranged from 560.7 to 592.9 K. Emergency core coolant (ECC) injection into the intact loop cold leg was initiated automatically at 14 s after experiment initiation. The specific objectives for LOCE L2-3 are stated in Reference 4.

The experiment prediction for LOCE L2-3 is best-estimate calculations of thermal-hydraulic responses in the LOFT system and was performed prior to conducting the experiment using the RELAP4/MOD6 computer code<sup>5</sup>. RELAP4/MOD6 is designed to calculate thermal-hydraulic responses, including fuel rod cladding temperatures, in a PWR during the blowdown and reflood phases of a loss-of-coolant transient. The test parameters used in the calculations are specified in Reference 4.

The experiment prediction and experimental data for LOCE L2-3 are compared in Section 2 on data plots with data from both sources overlaid. The experimental data shown are classified as qualified engineering units data, restrained data, not reviewed data, or computed data. These data classifications and the parameters compared are discussed in Section 2. Conclusions from the data comparisons and recommendations for future LOCE L2-3 posttest analysis work are presented in Section 3. Locations of the experimental measurements in the LOFT system are shown in figures in Appendix A. These figures are on foldout sheets that, when unfolded, allow the figures to be viewed alongside the data plots for reader clavenience.

## 2. DATA COMPARISONS

In the context of this report, the experimental and experiment prediction data are most easily compared on data plots showing corresponding data from both sources. (The reader is encouraged to consider the error bands listed in Reference 1 in relation to the experimental data.) The experimental data are classified on each plot as OEUD (qualified engineering units data), REST (restrained data), MORE (not reviewed data), or COPE (computed data). These classifications are defined in Section 2.1. The experimental parameters compared are discussed in Section 2.2

#### 2.1 Experimental Data Classifications

Individual plots of experimental data discussed in this report are classified in one of the following four categories.

2.1.1 <u>Qualified Engineering Units Data</u>. Qualified engineering units data (QEUD) must meet the following criteria:

- (1) Have had all calibration corrections applied
- (2) Have been compared with independent data and found to agree during the period of interest within specified uncertainty limits
- (3) Have been verified to represent the parameter being measured.

Analytical use of the data is unrestricted for the specified time periods and within the defined uncertainty bands.

2.1.2 <u>Restrained Data</u>. Restrained data (REST) appear reasonable but cannot be classified QEUD because they neet either one of the following criteria:

- They are outside uncertainty bands established by reference measurements or derived from redundant measurements when such are available
- (2) There are no independent data available for applying required calibration corrections during particular time intervals, and there are either no redundant measurements with which to compare the data or the data are outside uncertainty bands derived by such comparison when redundant measurements are available.

Restricted data used in numerical analyses are constrained by their restrictive statements included on the data plots.

2.1.3 <u>Not Reviewed Data</u>. Not reviewed data (NORE) have not been reviewed by the Data Integrity Review Committee (DIRC) (this definition is for internal recordkeeping only).

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2.1.4 <u>Computed Data</u>. Computed data (COPE) have been calculated from measured data using data processing subroutines. In the figures, the four letter acronym appears on the plot; the test data trace is indicated by open squares, while the experiment prediction trace is indicated by solid circles.

### 2.2 Experimental Parameters Compared

Many of the data plots for the parameters compared show a difference in initial conditions between the experiment prediction and experimental data. This difference is probably the result of one of two errors or a combination of both. The first probable error to be considered is that the initial test conditions were not exactly known prior to the experiment, and the second is that the physical system was not accurately described via input data to the RELAP4/MOD6 solution scheme. The figures showing data discussed in the following subsections are presented at the end of Section 2.

2.2.1 <u>Fluid Density</u>. Figures 1 through 4 show that the trends in the density behavior were well predicted. The intermittent slug flow past the densitometer in the intact loop cold leg upstream of the ECC injection point was not predicted, as can be seen in Figure 3.

2.2.2 <u>Differential Pressure</u>. Figures 5 through 11 show comparisons between predicted and measured differential pressures. In general, the trends of the experimental data were well predicted. The initial differential pressure across the pumps (see Figures 8, 10, and 11) and steam generator (see Figure 9) in the intact loop reflect an adjustment due to the elevation head between the taps. The experiment prediction data cannot be corrected for this transducer calibration effect, and consequently, do not agree with the data before the start of the transient.

2.2.3 <u>ECC Flow Rate</u>. Figures 12 through 14 show comparisons between predicted and measured ECC volumetric flow rates. The trends of the experimental data were well predicted.

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2.2.4 <u>Liquid Level</u>. The liquid level comparisons are shown in Figures 15 through 17. The trends in the experimental data were well predicted with the exception of steam generator secondary level. This is probably due to a misstatement of either the steam generator secondary feed or steam flow boundary conditions.

2.2.5 <u>Mass Flow Rates</u>. Figures 18 through 23 show comparisons between predicted and measured mass flow rates. The predictions are within the error bands of the experimental data for the majority of the transient.

2.2.6 <u>Momentum Flux</u>. Figures 24 through 31 show comparisons between predicted and measured momentum fluxes. With the exception of the broken loop cold leg, the prediction is for the most part in good agreement with the trends of the experimental data.

2.2.7 <u>Pressure</u>. Figures 32 through 47 show comparisons between the predicted and measured pressures. The pressures in the primary system were well predicted. The pressure in the blowdown suppression tank (see Figure 44) and steam generator secondary side (see Figure 45) were not well predicted.

2.2.8 <u>Pump Speed</u>. Figure 48 shows a comparison between predicted and measured pump speeds. The agreement is satisfactory.

2.2.9 <u>Fluid Velocity</u>. Figures 49 through 52 show comparisons between predicted and measured fluid velocities. The velocity in the broken loop cold leg (see Figure 49) was not well predicted; however, the trend of the velocity in the broken loop hot leg was well predicted.

The velocity in the intact loop hot and cold legs are in good agreement with the predictions. Since the velocity measurement transducers are unidirectional, it is believed that the experimental data shown in Figure 52 should be inverted from 8 to 30 s which would be in agreement with the predictions. This is substantiated by the bidirectional momentum flux measurements shown in Figure 27.

995 312

2.2.10 <u>Fluid Temperature</u>. Figures 53 through 89 show comparisons between predicted and measured fluid temperatures. In general, the trends of the predictions and the experimental data are in good agreement.

2.2.11 <u>Fuel Rod Cladding Temperature</u>. Figures 90 through 105 show comparisons between predicted and measured fuel rod cladding temperatures. The prediction exhibits early departure from nucleate boiling relative to the experimental data, and the rewet, which occurs early in the transient, is not predicted by the code. Considering the general good agreement of other experimental measurements, it is surprising that the cladding temperatures are not in better agreement. This may be an indication of a heat transfer problem in the code.

LTR 20-104



Fig. 1 Comparison of predicted and measured average density in broken loop cold leg (DE-BL-1).





LTR 20-104



Fig. 3 Comparison of predicted and measured average density in intact loop cold leg (DE-PC-1).



Fig. 4 Comparison of predicted and measured average density in intact loop hot leg (DE-PC-2).



Fig. 5 Comparison of predicted and measured differential pressure across 14- to 5-in. reducer (PdE-BL-1).



Fig. 6 Comparison of predicted and measured differential pressure across pump simulator (PdE-BL-5).

LTR 20-104



Fig. 7 Comparison of predicted and measured differential pressure across steam generator simulator (PdE-BL-7).



Fig. 8 Comparison of predicted and measured differential pressure across pumps (PdE-PC-1).



Fig. 9 Comparison of predicted and measured differential pressure across intact loop steam generator (PdE-PC-2).



Fig. 10 Comparison of predicted and measured differential pressure across Pump 1 (PdE-PC-9).

## POOR ORIGINAL LTR 20-104 0.15 Experimental data are good for initial conditions only; magnitude questionable after t = 0. • RELAP4/MOD6 Data Experimental Data (REST) WP.A 0 10 PRESS 0.05 FERENTIAL Pla 0.00 0141 -0.05 10 20. -10 0 30. 40. TIME AFTER RUPTURE (SECONDS)

Fig. 11 Comparison of predicted and measured differential pressure across Pump 2 (PdE-PC-10).



Fig. 12 Comparison of predicted and measured flow rate from accumulator (FT-P120-36-1).

995: 318

LTR 20-104



Fig. 13 Comparison of predicted and measured flow rate from lowpressure injection system pump (FT-P120-85).





995 319

LTR 20-104



Fig. 15 Comparison of predicted and measured liquid level in accumulator (LIT-P120-44).



Fig. 16 Comparison of predicted and measured liquid level in steam generator secondary side (LT-P004-8B).

LTR 20-104



Fig. 17 Comparison of predicted and measured liquid level in pressurizer Channel B (LT-P139-7).



Fig. 18 Comparison of predicted and measured mass flow in broken loop cold leg (FR-BL-1).

15

LTR 20-104







Fig. 20 Comparison of predicted and measured mass flow in intact loop cold leg (FR-PC-1).

LTR 20-104









LTR 20-104











Fig. 25 Comparison of predicted and measured momentum flux in broken loop hot leg (ME-BL-2).



Fig. 26 Comparison of predicted and measured momentum flux in intact loop cold leg (ME-PC-1).

LTR 20-104

326







Fig. 28 Comparison of predicted and measured momentum flux at instrument Stalk 1 (ME-1ST-1).



Fig. 29 Comparison of predicted and measured momentum flux in upper end box (ME-1UP-1).



Fig. 30 Comparison of predicted and measured momentum flux in upper end box (ME-3UP-1)  $\,$ 

LTR 20-104

995 328



Fig. 31 Comparison of predicted and measured momentum flux at instrument Stalk 2 (ME-2ST-1).



Fig. 32 Comparison of predicted and measured pressure in broken loop cold leg (PE-BL-1).



Fig. 33 Comparison of predicted and measured pressure in broken loop hot leg (PE-BL-2).



Fig. 34 Comparison of predicted and measured pressure in broken loop hot leg pump simulator outlet (PE-BL-3).
LTR 20-104

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Fig. 35 Comparison of measured and predicted pressure at steam generator simulator outlet (PE-BL-6).



Fig. 36 Comparison of predicted and measured pressure in broken loop cold leg spool piece midpoint (PE-BL-8).

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Fig. 37 Comparison of predicted and measured pressure in intact loop cold leg (PE-PC-1).



Fig. 38 Comparison of predicted and measured pressure in intact loop hot leg (PE-PC-2).







Fig. 40 Comparison of measured and predicted pressure at instrument Stalk 1 (PE-1ST-1A).

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Fig. 41 Comparison of predicted and measured pressure at instrument Stalk 1 (PE-1ST-3A).



Fig. 42 Comparison of predicted and measured pressure in upper end box (PE-1UP-1A).

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Fig. 43 Comparison of predicted and measured pressure at instrument Stalk 2 (PE-2ST-1A).



Fig. 44 Comparison of predicted and measured pressure in blowdown suppression tank (PE-SV-17).

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Fig. 45 Comparison of predicted and measured pressure in steam generator secondary side (PT-P004-10A).



Fig. 46 Comparison of predicted and measured pressure in accumulator (PT-P120-43).

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Fig. 47 Comparison of predicted and measured pressure at ECC cold leg injection point (PT-P120-61).





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Fig. 49 Comparison of predicted and measured average velocity in broken loop cold leg (FE-BL-1).



Fig. 50 Comparison of predicted and measured average velocity in broken loop hot leg (FE-BL-2).

LTR 20-104

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LTR 20-104



Fig. 53 Comparison of predicted and measured average coolant temperature in broken loop cold leg (TE-BL-1).





LTR 20-104

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Fig. 55 Comparison of predicted and measured average coolant temperature in intact loop cold leg (TE-PC-1).



Fig. 56 Comparison of predicted and measured average coolant temperature in intact loop hot leg (TE-PC-2).

LTR 20-104



Fig. 57 Comparison of predicted and measured coolant temperature on instrument Stalk 1 (TE-1ST-1).



Fig. 58 Comparison of predicted and measured coolant temperature on instrument Stalk 1 (TE-1ST-2).

LTR 20-104



Fig. 59 Comparison of predicted and measured coolant temperature on instrument Stalk 1 (TE-1ST-3).



Fig. 60 Comparison of predicted and measured coolant temperature on instrument Stalk 1 (TE-1ST-4).

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Fig. 61 Comparison of predicted and measured coolant temperature on instrument Stalk 1 (TE-1ST-5).





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Fig. 63 Comparison of predicted and measured coolant temperature on instrument Stalk 1 (TE-1ST-8).





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Fig. 65 Comparison of predicted and measured coolant temperature on instrument Stalk 1 (TE-1ST-11).



Fig. 66 Comparison of predicted and measured coolant temperature on instrument Stalk 1 (TE-1ST-12).

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Fig. 68 Comparison of predicted and measured coolant temperature on instrument Stalk 1 (TE-1ST-14).

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Fig. 69 Comparison of predicted and measured coolant temperature on instrument Stalk 2 (TE-2ST-1).





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Fig. 71 Comparison of predicted and measured coolant temperature on instrument Stalk 2 (TE-2ST-3).







Fig. 73 Comparison of predicted and measured coolant temperature on instrument Stalk 2 (TE-2ST-7).



Fig. 74 Comparison of predicted and measured coolant temperature on instrument Stalk 2 (TE-2ST-9).

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Fig. 75 Comparison of predicted and measured coolant temperature on instrument Stalk 2 (TE-2ST-10).





LTR 20-104



Fig. 77 Comparison of predicted and measured coolant temperature on instrument Stalk 2 (TE-2ST-14).



Fig. 78 Comparison of predicted and measured coolant temperature in lower end box (TE-1LP-1).

LTR 20-104









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Fig. 81 Comparison of predicted and measured coolant temperature in upper end box (TE-1UP-1).



Fig. 82 Comparison of predicted and measured coolant temperature on drag disc-turbine transducer FE-1UP-1 (TE-1UP-5).

LTR 20-104



Fig. 83 Comparison of predicted and measured coolant temperature in upper end box (TE-2UP-1).



Fig. 84 Comparison of predicted and measured coolant temperature in upper end box (TE-3UP-1).

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Fig. 85 Comparison of predicted and measured coolant temperature on drag disc-turbine transducer FE-3UP-1 (TE-3UP-5).



Fig. 86 Comparison of predicted and measured coolant temperature in upper end box (TE-4UP-1).



Fig. 87 Comparison of predicted and measured coolant temperature in upper end box (TE-5UP-1).



Fig. 88 Comparison of predicted and measured coolant temperature on drag disc-turbine transducer FE-5UP-1 (TE-5UP-9).



Fig. 89 Comparison of predicted and measured coolant temperature in upper end box (TE-6UP-1).



Fig. 90 Comparison of predicted and measured cladding temperature in fuel Assembly 5 (TE-5F4-15).

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Fig. 93 Comparison of predicted and measured cladding temperature in fuel Assembly 5 (TE-5J4-21).





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Fig. 95 Comparison of predicted and measured cladding temperature in fuel Essembly 5 (TE-5J4-26).









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#### IMAGE EVALUATION TEST TARGET (MT-3)





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Fig. 97 Comparison of predicted and measured cladding temperature in fuel Assembly 5 (TE-5F4-30).





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Fig. 99 Comparison of predicted and measured cladding temperature in fuel Assembly 5 (TE-5L6-30).



Fig. 100 Comparison of predicted and measured cladding temperature in fuel Assembly 5 (TE-5D6-32).

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Fig. 101 Comparison of predicted and measured cladding temperature in fuel Assembly 5 (TE-5L6-32).



Fig. 102 Comparison of predicted and measured cladding temperature in fuel Assembly 5 (TE-5D6-37).

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Fig. 105 Comparison of predicted and measured cladding temperature in fuel Assembly 5 (TE-5L6-39).

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#### 3. CONCLUSIONS AND RECOMMENDATIONS

The overall agreement between the experiment predictions and the experimental data is good, except for the fuel rod cladding thermal response and the performance of the steam generator secondary side.

It is recommended that future LOCE L2-3 posttest analysis activities be centered on gaining a better understanding of modeling rewet phenomena with the RELAP4/MOD6 heat transfer surface. In addition, better modeling of the performance of the steam generator secondary side is needed before modeling small break LOCEs.

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60

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### APPENDIX A

### EXPERIMENTAL MEASUREMENT LOCATIONS IN THE LOFT SYSTEM

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#### APPENDIX A

### EXPERIMENTAL MEASUREMENT LOCATIONS IN THE LOFT SYSTEM

Figure A-1 shows the intact loop and external reactor vessel instrumentation. Figure A-2 shows the broken loop instrumentation, Figures A-3 and A-4 show the internal reactor vessel instrumentation. Figure A-5 shows a core map. The core thermocouple identification number can be used with this map to locate a transducer in the core. The identification number is broken down as follows

Transducer location (inches from bottom of core)-Fuel assembly row ----Fuel assembly column -Fuel assembly number -Transducer type ---23 3B11



Fig. A-1 Intact loop measurement locations.

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tor vessel measurement locations.

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Section A-A



\*\* Station numbers are a dimensionless measure of relative constitution within the reactor wessel. They are assigned in increments of 2.54 centimeters with station 300.00 defined at the care barrel support ledge inside the reactor vessel flange.



Fig. A-3 F

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Station *	Fuel Assembly 1	Fuel Assembly 2	Fuel Assembly 3	Fuel Assembly 4	Fuel Assembly 5	Fuel Assembly 6	
290 -	O-TE-1UP-4		TE-3UP-4 8 TE-3UP-8 LE-3UP-1-1				
280 -	o-TE-1UP-3		0 TE-3UP-3 0 LE-3UP-1-2 TE-3UP-9				
270 -			0 TE-3UP-10				Nozzle
260 -			0-TE-3UP-11 LE-3UP-1-4				conterline
250 -		1	0-TE-3UP-12 LE-3UP-1-5				
240 -		1	LE-3UP-13	1			
230 -	0-TE-1UP-6	0 TE-2UP-4	C TE-3UP-14 LE-3UP-1-7 TE-3UP-6	0- TE-4UP-4		0 TE-6UP-4	
220 -			TE-3UP-15				
210 -	O- PE-1UP-1FF		TE-3UP-16 PE-3UP-1FF	1	FE-5UP-1 / TE-5UP-9 DTT'/ ME-5UP-1		
200 - D	0- TE-11)P-7	o-TE-2UP-5	ODT FE-3UP-7	o-TE-4UP-5	TE-5UP-1 TE-5UP-2	o TE-6UP-5	
190 —	ME-1UP-1	0-TE-2UP-1	ME-3UP-1	TE-4UP-1	TE-SUP-4	OTE-OUP-3	Top of upper
180 -	TE-1UP-1 TE-1UP-2	TE-2UP-2 TE-2UP-3	TE-3UP-1 TE-3UP-2	TE-4UP-3	TE-SUP-5 TE-SUP-6 TE-SUP-7 TE-SUP-8	TE-6UP-2 TE-6UP-1	tie plate
* St	ation numbers are	a dimensionless	s measure of			INEL-A-7373	

relative elevation within the reactor vessel. They are assigned in increments of 25.4 mm with station 300.00 defined at the core barrel support ledge inside the reactor vessel flange.

Fig. A-4 Reactor vessel upper plenum measurement locations.

71



Fig. A-5 Core map showing fuel rod position designations.