Florida Power
CORPORATION

September 24, 1979

File: 3-0-3-e

Director
Office of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Washington, DC 20555

Subject: Crystal River Unit 3
Docket No. 50-302
Operating License No. DPR-72

Dear Sir:

Enclosed are three (3) originals and forty (40) copies of Technical Specification Change Request Nos. 50 and 51, requesting amendment to Appendix A of Operating License No. DPR-72. As part of this request, proposed replacement pages for Appendix A are enclosed.

Change Request No. 50 is considered to be a Class III amendment per 10 CFR 170.22 as it involves a single safety issue. The licensing fee for this Change Request is waived by Mr. R. W. Reid's letter of July 23, 1979 as FPC has previously submitted an application for a license amendment concerning steam generator monitoring equipment prior to March 22, 1978, that has yet to be issued by the NRC. That application was submitted on March 17, 1978 and it is superseded by this Change Request.

FPC considers Change Request No. 51 to be a Class II amendment per 10 CFR 170.22 as it is administrative in nature.

A licensing fee in the amount of one thousand two hundred dollars ($1,200.00) and a signed copy of Certificate of Service for Technical Specification Change Request Nos. 50 and 51 to the Chief Executive of Citrus County, Florida, are enclosed.

Very truly yours,

FLORIDA POWER CORPORATION

W. P. Stewart

W. P. Stewart
Manager
Nuclear Operations

WPSekcM10(D6)

Attachments

cc: Office of Inspection and Enforcement
U.S. Nuclear Regulatory Commission
Suite 3100
101 Marietta Street
Atlanta, GA 30303

STATE OF FLORIDA

COUNTY OF PINELLAS

W. P. Stewart states that he is the Manager, Nuclear Operations, of Florida Power Corporation; that he is authorized on the part of said company to sign and file with the Nuclear Regulatory Commission the information attached hereto; and that all such statements made and matters set forth therein are true and correct to the best of his knowledge, information and belief.

W. P. Stewart

W. P. Stewart

Subscribed and sworn to before me, a Notary Public in and for the State and County above named, this 24th day of September, 1979.

Catherine S. Cameron

Notary Public

Notary Public, State of Florida at Large,
My Commission Expires: August 8, 1983

CameronNotary 3(D12)

UNITED STATES OF AMERICA
NUCLEAR REGULATORY COMMISSION

IN THE MATTER OF	)	
	)	DOCKET NO. 50-302
FLORIDA POWER CORPORATION	)	

CERTIFICATE OF SERVICE

W. P. Stewart deposes and says that the following has been served on the Chief Executive of Citrus County, Florida by deposit in the United States mail, addressed as follows:

Chairman, Board of County
Commissioners of Citrus County
Citrus County Courthouse
Inverness, Florida 32650

An original copy of Technical Specification Change Request Nos. 50 and 51, requesting amendment to Appendix A of Operating License No. DPR-72.

FLORIDA POWER CORPORATION

W. P. Stewart
W. P. Stewart
Manager, Nuclear Operations

SWORN TO AND SUBSCRIBED BEFORE ME THIS 24th DAY OF SEPTEMBER, 1979.

Catherine J. Comeroe
Notary Public

Notary Public State of Florida at Large
My Commission expires: August 8, 1983

(NOTARIAL SEAL)

Cert.Serv.(D12)

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Technical Specification Change Request No. 50 [Appendix A]

Replace License pages 4 and 5 and Appendix A pages VI, 3/4 7-10, 3/4 7-11, 3/4 7-12 and B 3/4 7-3 with the attached revised License pages 4 and 5 and Appendix A pages VI, 3/4 7-10 and B 3/4 7-3, respectively.

Proposed Changes

Delete the Secondary Water Chemistry Technical Specification (3/4.7.1.6) and incorporate a Secondary Water Chemistry Monitoring Program License Condition (2.C.(9)).

Reason for Proposed Change

In Mr. R. W. Reid's letter of July 23, 1979 (received July 26, 1979), Florida Power Corporation was requested to submit a proposed amendment to our license to delete our existing Technical Specifications on secondary water chemistry and to incorporate the requirements of the Model License Condition enclosed in the letter into the body of our license within 60 days (September 24, 1979). This proposed change is in response to that request.

Safety Analysis Justifying the Proposed Change

The NRC has determined that, in a number of instances, the secondary water chemistry technical specifications have significantly restricted the operational flexibility of some plants with little or no benefit with regard to limiting corrosion of steam generator tubes. Based on this experience and the knowledge gained in recent years, a more effective approach is to institute a license condition that requires the implementation of a secondary water chemistry monitoring and control program containing appropriate procedures and administrative controls. The required program and procedures will be developed by Florida Power Corporation and thus will more readily account for site and plant specific factors that affect chemistry conditions in steam generators. In the NRC's view, such a license condition would provide assurance that Florida Power Corporation will denote proper attention to controlling secondary water chemistry, while also providing the needed flexibility to more effectively deal with any off-normal conditions that might arise. Moreover, the NRC has concluded that such a license condition, in conjunction with existing technical specifications on steam generator tube leakage and inservice inspection, will provide the most practical and comprehensive means of assuring that steam generator tube integrity will be maintained.

of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

2.C.(1) Maximum Power Level

Florida Power Corporation is authorized to operate the facility at a steady state reactor core power level not in excess of 2452 megawatts (100 percent of rated core power level).

2.C.(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 22, are hereby incorporated in the license. Florida Power Corporation shall operate the facility in accordance with the Technical Specifications.

2.C.(3) Florida Power Corporation shall not operate the reactor in operational Modes 1 and 2 with less than three reactor coolant pumps in operation until safety analyses for less than three pump operation have been submitted by the licensees and approval has been granted by the Commission by amendment to this license.

2.C.(4) Deleted (Amendment No. 20, 3 July 1977)

2.C.(5) Deleted

2.C.(6) Deleted (Amendment No. 21, 3 July 1977)

2.C.(7) Deleted

2.C.(8) Deleted (Letters to NRC dated 31 January 1977 and 10 May 1977, accepted by NRC 31 May 1977)

2.C.(9) The licensee shall implement a secondary water chemistry monitoring program to inhibit steam generator tube degradation. This program shall include:

1. Identification of a sampling schedule for the critical parameters and control points for these parameters;

2. Identification of the procedures used to quantify parameters that are critical to control points;

3. Identification of process sampling points;

4. Procedure for the recording and management of data;

5. Procedures defining corrective actions for off control point chemistry conditions, and

6. A procedure identifying the authority responsible for the interpretation of the data, and the sequence and timing of administrative events required to initiate corrective action.

D. Florida Power Corporation shall maintain in effect and fully implement all provisions of the NRC Staff-approved physical security plan, including changes made pursuant to the authority of 10 CFR 50.54(p). The approved security plan consists of the original plan entitled "Florida Power Corporation's Security Plan for Crystal River Unit 3 Nuclear Generating Plant" as revised and described in the following letters:

Revision	Date of FPC Letters
Original	October 12, 1973
1	December 21, 1973
2	February 18, 1976
3	March 19, 1976

Pursuant to 10 CFR Section 2.790(d), the security plan is being withheld from public disclosure because it is deemed to be commercial or financial information within the meaning of 10 CFR Section 9.5(a)(4) and subject to disclosure only in accordance with 10 CFR Section 9.12.

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LIMITING CONDITIONS FOR OPERATION AND SURVEILLANCE REQUIREMENTS

DELETED

positive reactivity effects of the Reactor Coolant System cooldown associated with the blowdown, and 2) limit the pressure rise within containment in the event the steam line rupture occurs within containment. The OPERABILITY of the main steam isolation valves within the closure times of the surveillance requirements are consistent with the assumptions used in the safety analyses.

3/4 7.2 STEAM GENERATOR PRESSURE/TEMPERATURE LIMITATION

The limitation on steam generator pressure and temperature ensures that the pressure induced stresses in the steam generators do not exceed the maximum allowable fracture toughness stress limits. The limitations of 110°F and 237 psig are based on a steam generator RT_{NDT} of 40°F and are sufficient to prevent brittle fracture.

3/4 7.3 CLOSED CYCLE COOLING WATER SYSTEMS

The OPERABILITY of the closed cycle cooling water system ensures that sufficient cooling capacity is available for continued operation of safety related equipment during normal and accident conditions. The redundant cooling capacity of this system, assuming a single failure, is consistent with the assumptions used in the safety analyses.

Technical Specification Change Request No. 51 (Appendix A)

Replace page 6-3 with the revised page 6-3.

Proposed Change

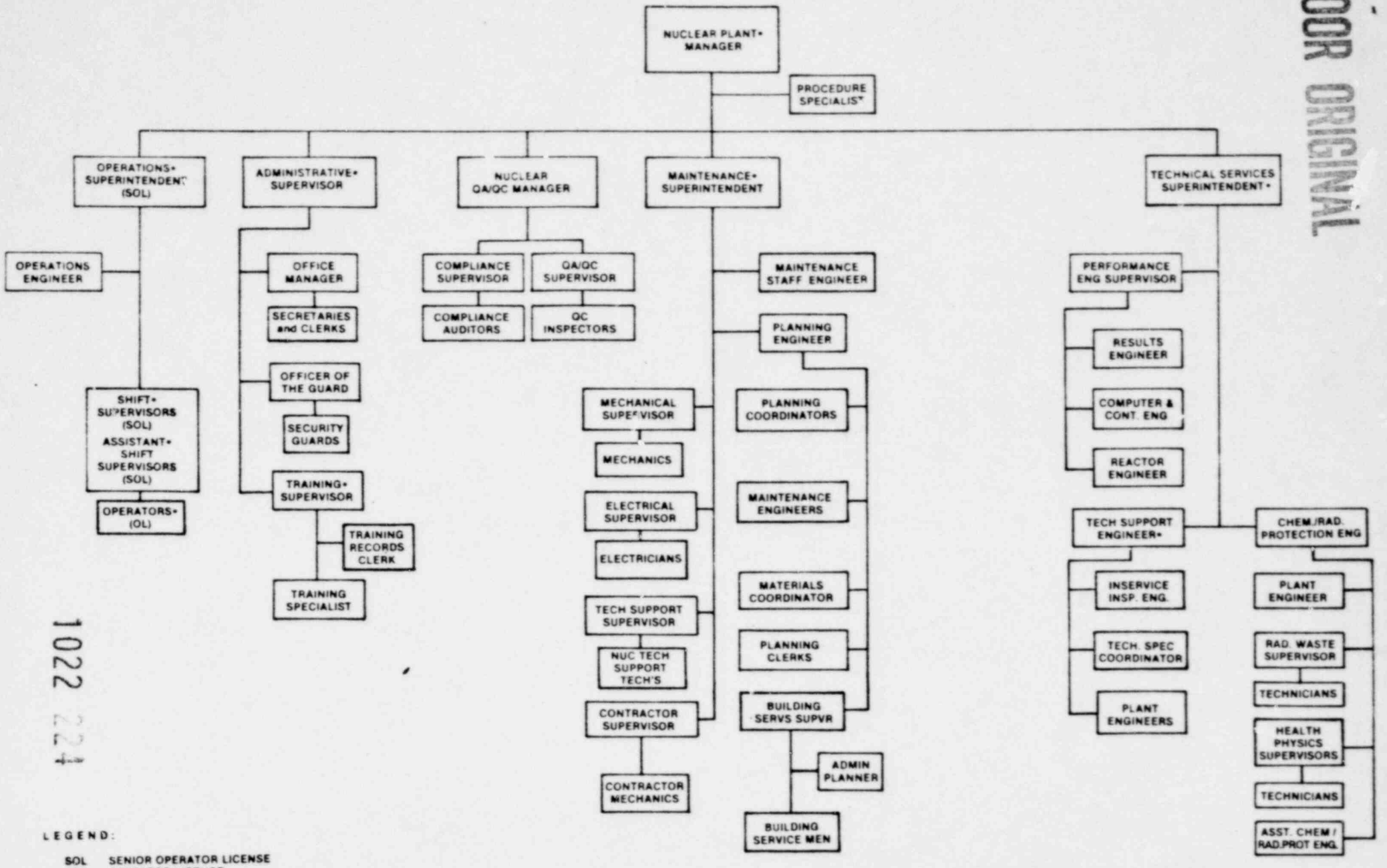
Realignment of the QA/QC group at Crystal River Unit 3. This organizational change consolidates the Compliance Section and the QC Inspectors under a new position, Nuclear QA/QC Manager, which reports directly to the Nuclear Plant Manager.

Reason for the Proposed Change

On March 23, 1979, Florida Power Corporation met with NRC Region II personnel in Atlanta to discuss their concerns about the effectiveness of our operational quality assurance program and management control system. As a result of this meeting and the QA/QC Audit performed by Region II, Florida Power Corporation, in our response of June 15, 1979, identified the attached organizational restructure as an essential element in developing an upgraded Quality Control Program at CR#3. This realignment is necessary to create a higher degree of independence for the QA/QC function, as well as to more clearly define the responsibilities of the QA/QC organization at CR#3.

Safety Evaluation Justifying the Proposed Change

No unreviewed safety question is involved with this change. As identified by Region II, this realignment will ensure that CR#3 is operated in a safe manner in accordance with NRC rules and regulations.



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LEGEND:

SOL SENIOR OPERATOR LICENSE
OL OPERATOR LICENSE
* INDICATES POSITION HAVING FIRE PROTECTION RESPONSIBILITY

FIGURE 6.2-2 FACILITY ORGANIZATION