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Portland General Electric Company

Charles Goodwin, Jr. Assistant Vice President

July 17, 1979

Trojan Nuclear Plant
Docket 50-344
License NPF-1

Mr. R. H. Engelken, Director
U. S. Nuclear Regulatory Commission
Region V
Suite 202, Walnut Creek Plaza
1990 N. California Blvd.
Walnut Creek, CA 94596

Dear Sir:

Attached please find the supplemental information as indicated in my letter to you dated May 18, 1979, which submitted our response to Item 13 of IE Bulletin 79-06A, Revision 1. The attached table summarizes the description and implementation schedule of design modifications at the Trojan Nuclear Plant as a result of the preliminary review of the TMI accident. The implementation schedule described in the table represents our best estimation to date based on available material delivery dates and the Trojan plant operational schedule.

A review of the Administrative Section of the Trojan Technical Specifications is currently underway. Some possible organizational changes have been identified and are being further reviewed and evaluated. This additional review and evaluation should be completed within 60 days. Should some of these organizational changes be desirable or necessary, a request for a license amendment will be submitted within 30 days after completion of the review or by mid-October 1979. Our survey and evaluation of management and technical resources in response to Harold Denton's letter of June 29, 1979 will supplement our current review and will be submitted to you by July 30, 1979.

In addition, a sufficiently high priority has been given for reviewing the Radiological Emergency Response Plan (RERP) in light of the TMI

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accident. We are currently working with the State of Oregon on the Oregon Emergency Operations Plan which will be submitted to the NRC for concurrence this fall. We also hope to have amended the Trojan RERP by this fall to incorporate those TMI-related changes presently identified.

Sincerely,

C. Goodwin, Jr.

C. Goodwin, Jr.
Assistant Vice President
Thermal Plant Operation and
Maintenance

CG/KM/4sb8A16
Attachment

c: Mr. Lynn Frank, Director
State of Oregon
Department of Energy

IMPLEMENTATION SCHEDULE OF DESIGN MODIFICATIONS IDENTIFIED
AS A RESULT OF PRELIMINARY REVIEW OF THE TMI ACCIDENT

Trojan Design Modification	Description of Design Modification	Status of Design Review	Implementation Schedule
1. SI Initiation Logic Change	Disconnect the input relays of the pressurizer level bistables at the universal logic boards in the SSPS cabinet and change the input relay connectors of the pressurizer pressure bistables to produce an actuation signal from a two-out-of-three logic.	Completed.	Modifications were implemented and the system is currently in service.
2. Containment Sump Discharge	Change the control circuits of Containment isolation valves MO-4180 and CV-4181 on the Containment sump line (FSAR Figure 11.2-14). This is to eliminate, following a Containment isolation signal reset, the automatic pumping of waste water out of Containment if a high Containment sump level exists. In addition, a process radiation monitor will be installed on the Containment sump discharge line to monitor activity level of the liquid to be transferred from the Containment sump to the Auxiliary Building dirty waste drain tank or the clean waste receiving tank.	Preliminary design review stage.	Modification will be implemented before the startup of Cycle 3 operation(1).

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3. PERM Range	Increase the upper ranges of the Auxiliary Building and main condenser air ejector Process Effluent Radiation Monitors (PERM-2 and PERM-6, respectively). The upper range of PERM-2 and PERM-6 will become 3×10^2 μCi/cc (Xe-133).	Detailed design review stage.	Modification will be implemented before the startup of Cycle 3 operation(1).
4. Redesign of Containment Isolation Valve Response on Resetting Containment Isolation	Modify the control circuits of Containment isolation valves which automatically open, if previously in an open position, after the Containment isolation signal is reset. Approximately 21 valves will be modified to prevent automatic opening after Containment isolation signal reset. With present Trojan design, one of the redundant series Containment isolation valves in each line remains closed until opened by separate operator action, except steam generator blowdown sample line valves.	Prelimiary design review stage.	Modification will be implemented before the startup of Cycle 3 operation(1).

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5. Recycle Connection From Waste Gas System to Containment	Provide a cross connect return line from the waste gas decay tanks to the Containment Building. Waste gas will be discharged back to Containment from any waste gas decay tank or pumped back directly from the waste gas compressors by manual alignment of the valve and spool piece which will be located on the return line.	Preliminary design review stage.	Modification will be implemented before the startup of Cycle 4 operation(2).
6. Low Suction Pressure Protection for Auxiliary Feedwater System (AFS) Pumps	Install a low suction pressure trip switch for each auxiliary feedwater pump, or an equivalent protective device.	Preliminary design review stage.	Modification will be implemented before the startup of Cycle 3 operation(1).
7. Electric Motor-Driven AFS Pump	Add an electric motor-driven AFS pump which is capable of delivering 1000 gpm at a head of 3400 ft. The new pump will take suction from the condensate storage tank and provide flow to an existing AFS pump discharge line.	Detailed design review stage. Piping is installed.	Modification will be implemented before the startup of Cycle 3 operation(1).

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8. Status Indication of AFS Manual Valves	Install position indicators to the AFS manual valves that isolate the motor-operated control valves. Indication of these valve positions will be located at panel C05 in the control room and locally at panel C160 in the form of red and green indicating lights.	Preliminary design review stage.	Modification will be implemented before the startup of Cycle 3 operation(1).
9. Independent Cooling Water Supply for Turbine-Driven AFS Pump	Add a self-cooling water supply to the AFS pump lube oil and bearing heat exchangers to ensure that pump operation will be independent of the Service Water System. The self-cooling water will be taken from the AFS pump discharge.	Detailed design review stage.	Modification will be implemented before the startup of Cycle 3 operation(1).

(1) Cycle 3 operation is currently expected to commence in the first half of 1980 (February thru May).
(2) Cycle 4 operation is expected to commence in the spring of 1981.

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