



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

WISCONSIN ELECTRIC POWER COMPANY  
DOCKET NO. 50-256  
POINT BEACH NUCLEAR PLANT UNIT NO. 1  
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 39  
License No. DPR-24

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - B. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - C. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - D. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by adding the following paragraph 3.H.:
  - H. The licensee may proceed with and is required to complete the modifications identified in Paragraphs 3.1.1 through 3.1.33 of the NRC's Fire Protection Safety Evaluation Report (SER) for the facility dated August 2, 1979. These modifications shall be completed as specified in Table 3.1 of the SER or supplements thereto.

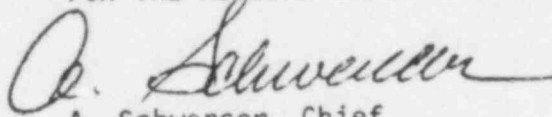
The licensee is required to implement and maintain the administrative controls identified in Section 6 of the NRC's Fire Protection Safety Evaluation Report on the Facility dated August 2, 1979 and supplements thereto.

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3. This license amendment becomes effective as of August 2, 1979\*.

FOR THE NUCLEAR REGULATORY COMMISSION



A. Schwencer, Chief  
Operating Reactors Branch #1  
Division of Operating Reactors

Date of Issuance: August 2, 1979

\*Provided no hearing is requested under 10 CFR Part 2 paragraph 2.204.

Amendment Pages for Operating License No. DPR-24

1. This amended license applies to the Point Beach Nuclear Plant Unit No. 2, a closed cycle, pressurized, light water moderated and cooled reactor, and associated steam generators and electric generating equipment (the facility). The facility is located on the licensee's Point Beach site, in the Town of Two Creeks, Manitowoc County, Wisconsin, and is described in the "Final Safety Analysis Report", as supplemented and amended.
2. Subject to the conditions and requirements incorporated here in the Commission hereby licenses
  - A. Pursuant to Section 104b of the Act and 10 CFR Part 50, "Licensing of Production and Utilization Facilities", to possess, use, and operate the facility at the designated location on the Point Beach site in accordance with the procedures and limitations set forth in this license;
  - B. Pursuant to the Act and 10 CFR Part 70, to receive, possess and use at any time special nuclear material as reactor fuel, in accordance with the limitations for storage and amounts required for reactor operation, as described in Final Facility Description and Safety Analysis Report, as supplemented and amended as of March 17, 1976;
  - C. Pursuant to the Act and 10 CFR Parts 30, 40 and 70 to receive, possess and use at any time any byproduct, source, and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;
  - D. Pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess and use in amounts as required any byproduct, source of special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components;
  - E. Pursuant to the Act and 10 CFR Parts 30 and 70, to possess such byproduct and special nuclear materials as may be produced by the operation of the facility, but not to separate such materials retained within the fuel cladding.

3. This amended license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations: 10 CFR Part 20, Section 30.34 of 10 CFR Part 30, Section 40.41 of 10 CFR Part 40, Sections 50.54 and 50.59 of 10 CFR Part 50, and Section 70.32 of 10 CFR Part 70; and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified below:

A. Maximum Power Levels

The licensee is authorized to operate the facility at reactor core power levels not in excess of 1518 megawatts thermal.

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 39 are hereby incorporated in the license. The licensee shall operate the facility in accordance with Technical Specifications.

C. Report

The licensee shall make certain reports in accordance with the requirements of the Technical Specifications.

D. Records

The licensee shall keep facility operating records in accordance with the requirements of the Technical Specifications.

E. Spent Fuel Pool Modification

The licensee is authorized to modify the spent fuel storage pool to increase its storage capacity from 351 to 1502 assemblies as described in licensee's application dated March 21, 1978, as supplemented and amended. In the event that the on-site verification check for poison material in the poison assemblies discloses any missing boron plates, the NRC shall be notified and an on-site test on every poison assembly shall be performed.

- F. The licensee shall maintain in effect and fully implement all provisions of the Commission-approved physical security plan, including amendments and changes made pursuant to the authority of 10 CFR 50.54(p). The approved security plan documents withheld from public disclosure pursuant to 10 CFR 2.790(d), are collectively titled "Point Beach Nuclear Plant Modified Amended Security Plan", dated January 4, 1978, as revised September 25, 1978, February 2, and March 29, 1979.

G. Safety Injection Logic

The licensee is authorized to modify the safety injection actuation logic and actuation power supplies and related changes as described in licensee's application for amendment dated April 27, 1979, as supplemented May 7, 1979. In the interim period until the power supply modification has been completed, should any DC powered safety injection actuation channel be in a failed condition for greater than one hour, the unit shall thereafter be shutdown using normal procedures and placed in a block-permissive condition for safety injection actuation.

- H. The licensee may proceed with an is required to complete the modifications identified in Paragraphs 3.1.1 through 3.1.33 of the NRC's Fire Protection Safety Evaluation Report (SER) for the facility dated August 2, 1979. These modifications shall be completed as specified in Table 3.1 of the SER or supplements thereto.

The licensee is required to implement and maintain the administrative controls identified in Section 6 of the NRC's Fire Protection Safety Evaluation Report on the Facility dated August 2, 1979 and supplements thereto.

4. The issuance of this amended license is without prejudice to subsequent licensing action which may be taken by the Commission with regard to the on-going rulemaking hearing on the Interim Acceptance Criteria for Emergency Core Cooling Systems (Docket No. RM 50-1).

5. This amended license is effective as of the date of issuance, and shall expire at midnight on July 25, 2008.

FOR THE ATOMIC ENERGY COMMISSION

Original Signed By

A. Giambusso, Deputy Director  
for Reactor Projects  
Directorate of Licensing

Attachments:

1. Appendix A - Technical Specifications
2. Appendix B - Environmental Technical Specifications

Date of Issuance: March 8, 1973





UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

WISCONSIN ELECTRIC POWER COMPANY

DOCKET NO. 50-301

POINT BEACH NUCLEAR PLANT UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 44  
License No. DPR-27

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - B. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - C. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - D. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

Accordingly, the license is amended by adding the following paragraph 3.H.:

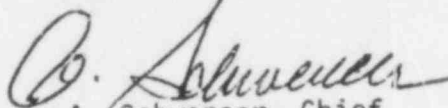
- H. The licensee may proceed with an is required to complete the modifications identified in Paragraphs 3.1.1 through 3.1.33 of the NRC's Fire Protection Safety Evaluation Report (SER) for the facility dated August 2, 1979. These modifications shall be completed as specified in Table 3.1 of the SER or supplements thereto.

The licensee is required to implement and maintain the administrative controls identified in Section 6 of the NRC's Fire Protection Safety Evaluation Report on the Facility dated August 2, 1979 and supplements thereto.



3. This license amendment becomes effective as of August 2, 1979\*.

FOR THE NUCLEAR REGULATORY COMMISSION



A. Schwencer, Chief  
Operating Reactors Branch #1  
Division of Operating Reactors

Date of Issuance: August 2, 1979

\*Provided no hearing is requested under 10 CFR Part 2 paragraph 2.204.

Amendment Pages for Operating License No. DPR-27

1. This amended license applies to the Point Beach Nuclear Plant Unit No. 2, a closed cycle, pressurized, light water moderated and cooled reactor, and associated steam generators and electric generating equipment (the facility). The facility is located on the licensee's Point Beach site, in the Town of Two Creeks, Manitowoc County, Wisconsin, and is described in the "Final Safety Analysis Report", as supplemented and amended.
2. Subject to the conditions and requirements incorporated here in the Commission hereby licenses
  - A. Pursuant to Section 104b of the Act and 10 CFR Part 50, "Licensing of Production and Utilization Facilities", to possess, use, and operate the facility at the designated location on the Point Beach site in accordance with the procedures and limitations set forth in this license;
  - B. Pursuant to the Act and 10 CFR Part 70, to receive, possess and use at any time special nuclear material as reactor fuel, in accordance with the limitations for storage and amounts required for reactor operation, as described in Final Facility Description and Safety Analysis Report, as supplemented and amended as of March 17, 1976;
  - C. Pursuant to the Act and 10 CFR Parts 30, 40 and 70 to receive, possess and use at any time any byproduct, source, and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;
  - D. Pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess and use in amounts as required any byproduct, source of special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components;
  - E. Pursuant to the Act and 10 CFR Parts 30 and 70, to possess such byproduct and special nuclear materials as may be produced by the operation of the facility, but not to separate such materials retained within the fuel cladding.

3. This amended license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations: 10 CFR Part 20, Section 30.34 of 10 CFR Part 30, Section 40.41 of 10 CFR Part 40, Sections 50.54 and 50.59 of 10 CFR Part 50, and Section 70.32 of 10 CFR Part 70; and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified below:

A. Maximum Power Levels

The licensee is authorized to operate the facility at reactor core power levels not in excess of 1518 megawatts thermal.

B. Technical Specifications

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C. Report

The licensee shall make certain reports in accordance with the requirements of the Technical Specifications.

D. Records

The licensee shall keep facility operating records in accordance with the requirements of the Technical Specifications.

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The licensee is required to implement and maintain the administrative controls identified in Section 6 of the NRC's Fire Protection Safety Evaluation Report on the Facility dated August 2, 1979 and supplements thereto.

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FOR THE ATOMIC ENERGY COMMISSION

Original Signed By

A. Giambusso, Deputy Director  
for Reactor Projects  
Directorate of Licensing

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