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TEXAS UTILITIES GENERATING COMPANY

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R. J. GARY

June 14, 1979 TXX-3001

Mr. Karl V. Seyfrit, Director U. S. Nuclear Regulatory Commission Region IV 611 Ryan Plaza Dr., Suite 1000 Arlington, Texas 76012

RIV Docket No. 50-445/IE Bulletin 79-07 50-446/IE Bulletin 79-07

COMANCHE PEAK STEAM ELECTRIC STATION
1981-83 2300 MW INSTALLATION
RESPONSE TO NRC
IE BULLETIN 79-07
FILE NO: 10115

Dear Mr. Seyfrit:

In response to IE Bulletin No. 79-07 we offer the following:

 None of the summation methods described in Item 1 of the bulletin were used for seismic analysis of safety-related piping for CPSES project.

The seismic analysis of safety-related piping was and is being performed in accordance with the requirements of USNRC Reg. Guide 1.92 revision 1 dated February 1976. The Combination of Modal Responses uses "GROUPING METHOD" (paragraph 1.2.1 of USNRC Reg. Guide 1.92) and the combination of effects due to three spatial components of an earthquake uses SRSS of "RESPONSE SPECTRA METHOD" (paragraph 2.1 of USNRC Reg. Guide 1.92). The ADLPIPE5, which is a 1977 version of ADLPIPE Computer Program is used to perform the seismic analysis of piping.

- A computer program listing is not required since none of the Item 1 techniques were or are being used.
- 3. The verification of the ADLPIPE5 program by Arthur O. Little was described on page 5 of Arthur O. Little's memorandum to the NRC dated April 9, 1979. The ADLPIPE5 program, which is a 1977 version of ADLPIPE Computer Program is used by the AE to perform the seismic analysis of piping.

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4. A plan of action is not required since none of the Item 1 techniques were or are being used.

In regard to piping system additions or modifications, these changes are subject to design review by the AE.

Should you have any questions, please advise.

Very truly yours,

P. J. Canu

RJG:dla

cc: U. S. Nuclear Regulatory Commission Office of Inspection and Enforcement Division of Reactor Operations Inspection Washington, D. C. 20555