

FGE



Portland General Electric Company
Trojan Nuclear Plant
P O Box 439
Rainier, Oregon 97048

June 26, 1979
BDW-575-79

Mr. R. H. Engelken, Director
Nuclear Regulatory Commission, Region V
1990 North California Blvd.
Walnut Creek, California 94596

Dear Sir:

SUBJECT: Immediate Report of Reportable Occurrence

This letter is a followup to a report made by Mr. W. S. Orser, Engineering Supervisor, to Mr. M. H. Malmros, NRC Resident Inspector at 9:00 a.m., Tuesday, June 26, 1979 concerning a situation identified by the Westinghouse Safety Review Committee (see attached letter) and reported by Westinghouse to the NRC on June 21, 1979 under the provisions of 10CFR21. High energy line breaks inside containment may result in heatup of the steam generator (S/G) level measurement reference leg. This will cause the indicated S/G water level exceeding the actual S/G water level resulting in the S/G low-low level reactor trip protection signal being delayed. In the specific case of a feedwater line rupture inside containment, the S/G low-low trip is the primary protection signal for this accident.

Backup reactor trip protection is available from S/G low level coincident with feed flow - steam flow mismatch, overtemperature ΔT , high pressurizer pressure, containment pressure and safety injection.

Prior to reactor startup we will change the S/G low-low level trip point from 5% to 15%. Additionally, operations personnel will be provided with appropriate instructions to compensate for containment temperature induced errors in the S/G level indication.

This occurrence will be reported to the Nuclear Regulator Commission as a Licensee Event Report in accordance with our established Technical Specification.

Sincerely,

B. D. Withers
Plant Superintendent

JCP
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