



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
REGION II  
101 MARIETTA ST., N.W., SUITE 3100  
ATLANTA, GEORGIA 30303

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In Reply Refer To:

RII:JPO

50-413, 50-414

50-488, 50-489

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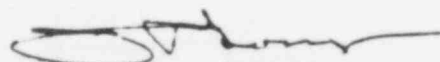
50-492, 50-493

Duke Power Company  
Attn: L. C. Dail, Vice President  
Design Engineering  
P. O. Box 33189  
Charlotte, North Carolina 28242

Gentlemen:

IE Bulletin 79-14 is revised to limit the scope of work required. The changes are indicated on the enclosed replacement page for the bulletin. If you desire additional information regarding this matter, please contact this office.

Sincerely,

  
James P. O'Reilly  
Director

Enclosure:  
Page 2 of 3 to IE Bulletin  
No. 79-14, Rev. 1

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Duke Power Company

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cc w/encl:

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Post Office Box 223  
Clover, South Carolina 29710

J. T. Moore, Project Manager  
Post Office Box 422  
Gaffney, South Carolina 29340

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Action to be taken by Licensees and Permit Holders:

All power reactor facility licensees and construction permit holders are requested to verify, unless verified to an equivalent degree within the last 12 months, that the seismic analysis applies to the actual configuration of safety-related piping systems. The safety related piping includes Seismic Category I systems as defined by Regulatory Guide 1.29, "Seismic Design Classification" Revision 1, dated August 1, 1973 or as defined in the applicable FSAR.

The action items that follow apply to all safety related piping 2 1/2-inches in diameter and greater and to seismic Category I piping, regardless of size which was dynamically analyzed by computer. For older plants, where Seismic Category I requirements did not exist at the time of licensing, it must be shown that the actual configuration of safety-related systems, utilizing piping 2 1/2 inches in diameter and greater, meets design requirements.

Specifically, each licensee is requested to:

1. Identify inspection elements to be used in verifying that the seismic analysis input information conforms to the actual configuration of safety-related systems. For each safety-related system, submit a list of design documents, including title, identification number, revision, and date, which were sources of input information for the seismic analyses. Also submit a description of the seismic analysis input information which is contained in each document. Identify systems or portions of systems which are planned to be inspected during each sequential inspection identified in Items 2 and 3. Submit all of this information within 30 days of the date of this bulletin.
2. For portions of systems which are normally accessible\*, inspect one system in each set of redundant systems and all nonredundant systems for conformance to the seismic analysis input information set forth in design documents. Include in the inspection: pipe run geometry; support and restraint design, locations, function and clearance (including floor and wall penetration); embedments (excluding those covered in IE Bulletin 79-02); pipe attachments; and valve and valve operator locations and weights (excluding covered in IE Bulletin 79-04). Within 60 days of the date of this bulletin, submit a description of the results of this inspection. Where nonconformances are found which affect operability of any system, the licensee will expedite completion of the inspection described in Item 3.

\*Normally accessible refers to those areas of the plant which can be entered during reactor operations.

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