SECOND
ERRATA

NUREG-0515, MARCH 1979

SUPPLEMENT NO. 2
TO THE
SAFETY EVALUATION REPORT

ALLENS CREEK NUCLEAR GENERATING STATION
UNIT 1
DOCKET NO. 50-466

HOUSTON LIGHTING & POWER COMPANY

Page 2-5 Section 2.2.2 "Aircraft Risks"

Second paragraph, line 6, change "Wanne" to "Wayne."

Third paragraph, line 6, change "Wanne" to "Wayne."

Page 2-6 Table 2.1

First item, change "6-inch crude" to "10-inch crude."

Fifth item, change "Superior Oil Company" to "Peninsula Resource Corporation;" change "8-inch raw products line" to "8.6-inch natural gas line;" and change "600 psi" to "up to 50,000 ft^3 per day at up to 1,000 psi."

Sixth item, change "up to 400,000 ft^3 per day at 750 to 900 psi" to "up to 400,000,000 ft per day at up to 975 psi."

Page 2-17 Line 16

Change "Amendment 52" to "Amendment 53."

First full paragraph, lines four & five and lines 13 & 14

Change "seismic Category I" to "seismically designed."

Page 3-9 Section 3.10 "Seismic Qualification of Category I Instrumentation and Electrical Equipment," line 9

Change "However, the applicant has provided a commitment that all seismic Category I equipment used in the Allens Creek plant will meet the requirements of IEEE 344-1975 and Regulatory Guide 1.100, Revision 1" to "However, the applicant has provided a commitment that this equipment

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will meet the requirements of IEEE 344-1975 and Regulatory Guide 1.100, Revision 1, unless qualified by other means found acceptable by the staff during the operating license stage of review."

Page 4-1 Section 4.2.1(1), "Description," line 9

Change 2.52 to 2.50.

Page 4-11 Second full paragraph, line 8

Change ". . . boiling water reactor . . ." to ". . . Boiling Water Reactor . . ."

Page 4-11 Line 15

Change "January 17, 1978" to "September 22, 1976."

Page 5-4 Section 5.2.3(1), "Compliance with Code Requirements," line 9

Change "Appendix B" to "Appendix G."

Page 5-5 Section 5.2.6, heading

Change "Protection" to "Detection."

Page 6-7 Line 2

Change "Since then we have advised the applicant that due to the conservative plant operating conditions, as indicated by the results of analyses of emergency core cooling system performance for Allens Creek as well as for other BWR 6/ Mark III plants, we believed that the assumption of one-percent metal-water reaction provides a sufficiently conservative basis for combustible gas control system design." to

"Also, standards for combustible gas control system in light water cooled power reactors was issued as a Commission regulation on November 10, 1978 as Section 50.44 to 10 CFR Part 50."

Line 12

Change "Since five times the metal-water reaction calculated to demonstrate compliance with Section 50.46 of 10 CFR Part 50 (0.17 percent as reported in Section 6.3.2 of this supplement) is 0.85 percent and since 0.00023 inches is only about 0.7 percent of the clad wall thickness of 0.034 inches, the use of one percent assures compliance with Section 50.44 of 10 CFR Part 50, "Standards for combustible gas control system in light water cooled power reactors."" to

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"The applicant has agreed to provide a commitment in Amendment 53 to the PSAR that the combustible gas control system will be designed for hydrogen released by 0.85 percent metal-water reaction, i.e., five times the 0.17 percent reported in Section 6.3.2 of this supplement for compliance with Section 50.46 of 10 CFR Part 50. This commitment is in accordance with Section 50.44 of 10 CFR Part 50, and therefore is acceptable."

Page 6-7 — Section 6.2.5, "Combustible Gas Control"

Remove the last three paragraphs of Section 6.2.5.

Add one paragraph: "The redundant recombiners will be permanently installed inside containment. This is permitted by the provisions of Regulatory Guide 1.7 and is acceptable. The applicant has committed to provide a testing program to demonstrate the operability of the recombiners. We will review the testing program and operating procedures at the operating license stage of review."

Page 6-14 — Section 6.5 "Engineered-Safety-Feature Atmosphere Cleanup System"

Second paragraph, line 5, omit "(Revision 2, March 1978)."

Page 8-1 — Section 8.1 "General," line 12

Change "Use of IEEE Std. 308-1971, Criteria for Class IE Electric Systems for Nuclear Power Generating Stations" to

"Criteria for Safety-Related Electric Power Systems for Nuclear Power Plants."

Page 8-3 — Section 8.3.1, "Alternating Current Standby Power Sources," last paragraph

Change "IEEE Std. 208-1974" to "IEEE Std. 308-1974."

Last paragraph

Second line, change ". . . separate ventilating fan for each room. . ." to ". . . ventilation system. . ."

Third line, change "There will be no common ventilation ducts for the battery rooms." to "The ventilation system includes a common collection header, exhaust fans and a common discharge header."

Page 8-4 — Section 8.3.2, last paragraph, second line

Change "IEEE Std. 208-1971" to "IEEE Std. 308-1971."

Page 9-1 — Section 9.2.1, third line

Change "Section 8.9.2.1" to "Section 9.2.1."

Page 9-1 — Section 9.2.2, first paragraph

First line, change "Amendment 37" to "Amendments 37 and 48."

Last line, change "The new decay heat load is that produced by one-quarter of a full core load after 80 hours plus four successive annual batch discharges." to

"The new decay heat load is the sum of that produced by one-quarter of a full core load (irradiated for four years) after 30 hours plus nine successive annual batch discharges."

Page 9-2 — Line 7

Change "Section 9.22" to "Section 9.2.2."

Page 9-3 — Section 9.5.1, first paragraph, lines 2 and 3

Change ". . . added a . . ." to ". . . provided clarification that with Amendment 22 it had added a . . ."

Page 9-5 — Section 9.5.4, second paragraph, fourth line

Change ". . . Engine Fuel 0.1 Storage. . ." to ". . . Engine Fuel Oil Storage. . ."

Page 11-4 — Table 11.2, line 8

Change "Xe-133 66 250 66 10 2000 2300 2300" to

"Xe-133 66 250 66 10 20,000 2300 23,000".

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