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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

JULY 20 1979

Docket No. 50-271

Mr. Robert H. Groce
Licensing Engineer
Yankee Atomic Electric Company
20 Turnpike Road
Westboro, Massachusetts 01581

Dear Mr. Groce:

RE: VERMONT YANKEE NUCLEAR POWER STATION

We are reviewing your submittals dated April 27 and May 4, 1979, in response to IE Bulletin 79-08. We have determined that the additional information requested in the enclosure is necessary in order to complete our safety evaluation.

We request that responses to the items in the enclosure be forwarded to this office within two weeks of your receipt of the enclosure, which was previously transmitted to you by telecopy. Please contact William F. Kane at (301) 492-7745 if you require additional discussions or clarification regarding the information requested.

Sincerely,

A handwritten signature in dark ink, appearing to read "T. Ippolito", written over the typed name.

Thomas A. Ippolito, Chief
Operating Reactors Branch #3
Division of Operating Reactors

Enclosure:
Request for Additional
Information

cc w/enclosure:
See next page

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Mr. Robert H. Groce
Yankee Atomic Electric Company

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cc:

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VERMONT YANKEE
REQUESTS FOR ADDITIONAL INFORMATION

IEB 79-08

Item No. 2

1. Your response indicates that containment isolation of the recirculation loop sample line, main steam line drain lines and reactor cleanup system does not comply with the requirements of IEB 79-08. This is because containment isolation of these lines is not initiated on all automatic initiations of safety injection, i.e., containment isolation of these lines is initiated on low-low water level but not on high drywell pressure. Please provide justification for not isolating these lines or prepare and implement the necessary changes to isolate these lines as required by the Bulletin. Provide a schedule for completion of the necessary design and/or procedural changes to comply with the requirements of IEB 79-08.

Item No. 4

1. Your response is incomplete. Describe all uses and types of vessel level indication for both manual and automatic initiation of safety systems.
2. Describe other instrumentation which the operator might have to determine changes in reactor coolant inventory, e.g., drywell high pressure, radioactivity levels, suppression pool high temperature, containment sump pump operation, etc.
3. Clarify your response to indicate whether operators have been instructed to utilize other available information to initiate safety systems. Provide your schedule for completion of this action.

Item No. 5

1. It is not clear from your response whether your review of procedures, with respect to the actions directed by items 5a and 5b, included all operating procedures and training instructions. Amend your response to clarify this point.
2. It is not clear from your response whether your review of operating procedures and training instructions, with respect to the actions directed by item 5b, included additional information and instructions to operators to not rely on vessel level indication alone for manual actions but to also examine other plant parameter indications in evaluating plant conditions. Amend your response to clarify this point.
3. Provide a schedule for any actions on item 5 that have not yet been completed.

Item No. 6

1. It is not clear from your response that safety-related valve positioning requirements were reviewed to ensure proper operation of engineered safety features. Please supplement your response to provide a commitment to conduct this review and a schedule for completion.
2. Please augment your response to indicate the extent to which position and locking device checks are performed for locked safety system valves.
3. Your response indicated that refinements to valve line-up procedures are being made and a procedure prescribing instrumentation rack valve line-ups is being developed. Please provide a schedule for the completion of these activities.
4. Your response did not clearly indicate that all accessible safety-related valves had been inspected to verify proper position. Nor was a schedule for performing the position verification for all safety-related valves provided. Please supplement your response to provide this information.

Item No. 7

1. Verify that the systems listed in part b of your response include all those systems designed to transfer potentially radioactive gases and liquids out of containment.
2. Item No. 7 of IEB 79-08 requires assurance that inadvertent transfer of radioactive gases and liquids be prevented on resetting engineered safety features instrumentation. In your response you stated that appropriate procedural precautions are used. Discuss these precautions further and indicate how they are used to prevent inadvertent transfer on reset. If such precautions are not yet available, provide the schedule for their completion.

Item No. 8

1. We understand from your response that operability tests are performed on redundant safety-related systems prior to removal of any safety-related system from service. Since you may be relying on prior operability verification within the current technical specification surveillance interval, operability should be further verified by at least a visual check of the system status to the extent practicable, prior to removing the redundant equipment from service. Please supplement your response to provide a commitment that you will revise your maintenance and test procedures to adopt this position.

Item No. 9

1. Provide the date on which you completed the actions required by item 9 of IEB 79-08.

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