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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

JULY 20 1979

Docket No. 50-333

Mr. George T. Berry
General Manager and Chief
Engineer
Power Authority of the
State of New York
10 Columbus Circle
New York, New York 10019

Dear Mr. Berry:

RE: JAMES A. FITZPATRICK NUCLEAR POWER PLANT

We are reviewing your submittal dated April 25, 1979 in response to
IE Bulletin 79-08. We have determined that the additional information
requested in the enclosure is necessary in order to complete our safety
evaluation.

We request that responses to the items in the enclosure be forwarded
to this office within two weeks of your receipt of the enclosure, which
was previously transmitted to you by teletcopy. Please contact William
F. Kane at (301) 492-7745 if you require additional discussions or
clarification regarding the information requested.

Sincerely,

Thomas A. Ippolito, Chief
Operating Reactors Branch #3
Division of Operating Reactors

Enclosure:
Request for Additional
Information

cc w/enclosure:
See next page

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7908080

Mr. George T. Berry
Power Authority of the State
of New York

JULY 20 1979

cc:

Mr. Vito J. Cassan
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George M. Wilverding, Licensing Supervisor
Power Authority of the State of New York
10 Columbus Circle
New York, New York 10019

FITZPATRICK

REQUESTS FOR ADDITIONAL INFORMATION

IEB 79-08

Item No. 1

1. Confirm that all licensed operators, plant management and supervisors with operational responsibilities have completed the review of TMI-2 events as required by item 1 of IEB 79-08 and that their participation has been documented in plant records.

Item No. 2

1. Your response is incomplete in that it does not indicate that you have reviewed procedures to assure initiation of containment isolation is in conformance with Item 2 of IEB 79-08. Verify that you have reviewed the required procedures or provide the schedule for completion of that review.

Item No. 4

1. Describe other instrumentation which the operator might have to determine changes in reactor coolant inventory, e.g., drywell high pressure, radioactivity levels, suppression pool high temperature, containment sump operation, etc.
2. Clarify your response to indicate whether operators have been instructed to utilize other available information to initiate safety systems. Provide your schedule for completion of this action.

Item No. 5

1. Your response to items 5a and 5b does not address training instructions. Amend your response to address this matter.
2. Your response to item 5a does not address the matter of unsafe plant conditions. Amend your response to address this matter.
3. It is not clear from your response whether your review of procedures with respect to the actions directed by item 5b included all applicable operating procedures and training instructions. Amend your response to clarify this point.
4. Provide a schedule for any actions on item 5 that have not been completed.

Item No. 6

1. Please augment your response to indicate the extent to which position and locking device checks are performed for locked safety system valves; i.e., clarify monthly surveillance of "certain critical valves".
2. It is not clear from your response that positive administrative controls have been implemented to assure that systems requiring retest are in fact retested prior to the need for their operability. Please clarify your response to provide assurance that safety related valves are returned to their correct positions following necessary manipulations.
3. Your response did not clearly indicate that all accessible safety-related valves had been inspected to verify proper position. Nor was a schedule for performing the position verification for all safety-related valves provided. Please supplement your response to provide this information.

Item No. 7

1. Provide your schedule for modification of procedures to prevent inadvertent transfer of radioactive liquids through the Reactor Coolant Sample Line on resetting of engineered safety features instrumentation.

Item No. 8

1. We understand from your response that operability tests are performed on redundant safety related systems prior to removal of any safety related system from service. Since you may be relying on prior operability verification within the current technical specification surveillance interval, operability should be further verified by at least a visual check of the system status to the extent practicable, prior to removing the redundant equipment from service. Please supplement your response to provide a commitment that you will revise your maintenance and test procedures to adopt this position.
2. It is not clear from your response that all involved reactor operational personnel in the oncoming shift are explicitly notified about the status of systems removed from or returned to service. Please indicate how this information is transferred at shift turnover.

Item No. 9

1. Your response is unacceptable. NRC notification is required whenever "the reactor is not in a controlled or expected condition of operation". Serious accidents may not be the only events which warrant notification. Further, you must commit to notification within one hour and to the establishment of an open continuous communications channel.